

JAN 15 2001  
LRN-01-0004



U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

**MONTHLY OPERATING REPORT  
HOPE CREEK GENERATING STATION UNIT 1  
DOCKET NO. 50-354**

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **December 2000** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **December 2000**.

Sincerely,

A handwritten signature in black ink that reads "D. F. Garchow".

D. F. Garchow  
Vice President - Operations

RAR  
Attachments

C Distribution

Handwritten initials "JES4" in black ink, located in the bottom right corner of the page.

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DOCKET NO.: 50-354  
 UNIT: Hope Creek  
 DATE: 01/08/01  
 COMPLETED BY: F. Todd  
 TELEPHONE: (856) 339-1316

Reporting Period December 2000

**OPERATING DATA REPORT**

**Design Electrical Rating (MWe-Net)**  
**Maximum Dependable Capacity (MWe-Net)**  
  
**No. of hours reactor was critical**  
**No. of hours generator was on line (service hours)**  
**Unit reserve shutdown hours**  
**Net Electrical Energy (MWH)**

<b>1067</b>		
<b>1031</b>		
<b>Month</b>	<b>Year-to-date</b>	<b>Cumulative</b>
<b>744</b>	<b>7720</b>	<b>104447</b>
<b>744</b>	<b>7259</b>	<b>102406</b>
<b>0</b>	<b>0</b>	<b>0</b>
<b>786803</b>	<b>7271736</b>	<b>103570335</b>

**UNIT SHUTDOWNS**

<b>NO.</b>	<b>DATE</b>	<b>TYPE F=FORCED S=SCHEDULED</b>	<b>DURATION (HOURS)</b>	<b>REASON (1)</b>	<b>METHOD OF SHUTTING DOWN THE REACTOR (2)</b>	<b>CORRECTIVE ACTION/ COMMENT</b>

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

DOCKET NO.: 50-354  
UNIT: Hope Creek  
DATE: 01/08/01  
COMPLETED BY: R. Ritzman  
TELEPHONE: (856) 339-1445

### **Summary Of Monthly Operating Experience**

- Hope Creek started the month at approximately 100% reactor power.
- Reactor power was reduced to approximately 87% on December 1 for turbine control valve testing and to support Salem Unit 2 rectifier maintenance. Power was returned to approximately 100% on December 2.
- On December 4, reactor power was reduced to approximately 89% for maintenance on the New Freedom Line and the Salem Unit 2 rectifier. Power was returned to 100% power on December 5 and remained there the balance of the month.
- At the end of December, Hope Creek completed 158 days of continuous on-line operation.

DOCKET NO.: 50-354  
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TELEPHONE: (856) 339-1445

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE HOPE CREEK GENERATING STATION**

**MONTH December 2000**

The following item completed during **December 2000** has been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluation showed that this item did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. This item did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

**Design Changes Summary of Safety Evaluation**

There were no reportable changes in this category implemented during December 2000.

**Temporary Modifications Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 2000.

**Procedures Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 2000.

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE HOPE CREEK GENERATING STATION – Cont'd**

**UFSAR Change Notices Summary of Safety Evaluations**

**HCN 00-057, Changes to the Description of the Suppression Pool Cooling Mode of the Residual Heat Removal System.** This UFSAR change notice revised the order of the steps that are used to initiate the suppression pool cooling mode of the Residual Heat Removal system. It also changes the position of the heat exchanger bypass valve from throttled to closed.

These changes to the step sequence and the position of the heat exchanger bypass valve do not impact the design function of the containment heat removal function. Since the design function is maintained, there is no change in dose consequences. This UFSAR change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

**HCN 00-061, Chemistry Clarifications.** This UFSAR change notice revised the section on reactor coolant pressure boundary material compatibility with reactor coolant to address hydrogen water chemistry. This portion of the UFSAR was inadvertently missed during the previous UFSAR change notices addressing hydrogen water chemistry. This UFSAR change notice also eliminated a statement about maintaining a spare bed of resin in the resin regeneration system for use during resin bed cleaning or regeneration.

This UFSAR change notice does not impact the operation of a safety system. Part of the change notice is administrative in that it maintains consistency within the UFSAR. The statement about maintaining a spare bed of resin in the resin regeneration system for use during resin bed cleaning or regeneration may be eliminated because there is no longer a need for a spare resin bed. This UFSAR change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

**Technical Specification Bases Change Summary of Safety Evaluations**

**Safety Evaluation H2000-059, Change to Include Definition of Rod Block Monitor Operability.** This Technical Specification Bases Change added a definition of the Local Power Range Monitor configuration that is required for the Rod Block Monitor to be operable.

The addition of this definition does not impact the ability of the Local Power Range Monitors or the Rod Block Monitor to perform its design function. This Technical Specification Design Bases change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.