

January 17, 2001

Mr. Craig G. Anderson  
Vice President, Operations ANO  
Entergy Operations, Inc.  
1448 S. R. 333  
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNITS 1 AND 2 - REVISIONS TO TECHNICAL SPECIFICATION PAGES RELATED TO AMENDMENT ON REPLACEMENT STEAM GENERATOR INSPECTION REQUIREMENTS (TAC NO. MA6341)

Dear Mr. Anderson:

On October 4, 2000, the Nuclear Regulatory Commission (NRC) issued Amendment No. 223 for Arkansas Nuclear One, Unit 2 (ANO-2). The amendment revised selected steam generator inspection requirements to account for changes associated with replacement of the original steam generators.

Since issuance of the amendment, Mr. Dennis Boyd of your staff indicated that two of the technical specification (TS) pages issued in Amendment No. 223 do not list (and cross out) all of the previous amendment numbers associated with those pages, at the bottom right corner of those pages. The Federal Register notices regarding Amendment No. 223 did not identify these previous amendment numbers, nor did the NRC staff review those numbers in approving Amendment No. 223. We have reviewed these items and are hereby revising TS page 3/4 4-6 to list previous amendment numbers 210 and 217, and TS page 3/4 4-7 to list previous amendment number 187. No other changes have been made to these pages.

Enclosed are revised TS pages 3/4 4-6 and 3/4 4-7. We regret any inconvenience that this may have caused you.

Sincerely,

/RA/

Robert A. Gramm, Chief, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-368  
Enclosure: As stated  
cc w/encl: See next page

DISTRIBUTION:

PUBLIC  
PDIV-1 r/f  
RidsNrrDripRtsb (WBeckner)  
RidsNrrDlpmPdiv (SRichards)  
RidsNrrDlpmLpdiv1 (RGramm)

RidsNrrPMTALexion  
RidsNrrLADJohnson  
RidsRgn4MailCenter (KBrockman)  
RidsAcrcAcnwMailCenter

G. Hill(4)  
L.Hurley,RIV  
D.Bujoi,RIV  
RidsOgcRp

Accession No.:

\*see previous concurrence

OFFICE	PDIV-1/PM	PDIV-1/LA	OGC	PDIV-1/SC
NAME	TAlexion: <i>TWA</i>	DJohnson <i>dly</i>	RWeisman*	RGramm <i>lb</i>
DATE	01/11/01	1/11/01	01/07/01	1/12/01

DOCUMENT NAME: G:\PDIV-1\ANO2\CORA6341.wpd

OFFICIAL RECORD COPY

Arkansas Nuclear One

cc:

Executive Vice President  
& Chief Operating Officer  
Entergy Operations, Inc.  
P. O. Box 31995  
Jackson, MS 39286-1995

Vice President, Operations Support  
Entergy Operations, Inc.  
P. O. Box 31995  
Jackson, MS 39286-1995

Director, Division of Radiation  
Control and Emergency Management  
Arkansas Department of Health  
4815 West Markham Street, Slot 30  
Little Rock, AR 72205-3867

Wise, Carter, Child & Caraway  
P. O. Box 651  
Jackson, MS 39205

Winston & Strawn  
1400 L Street, N.W.  
Washington, DC 20005-3502

Manager, Rockville Nuclear Licensing  
Framatome Technologies  
1700 Rockville Pike, Suite 525  
Rockville, MD 20852

Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
P. O. Box 310  
London, AR 72847

Regional Administrator, Region IV  
U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011-8064

County Judge of Pope County  
Pope County Courthouse  
Russellville, AR 72801

REACTOR COOLANT SYSTEM

STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

---

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1,2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing Tavg above 200°F.

SURVEILLANCE REQUIREMENTS

---

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

NOTE: The requirements for inservice inspection do not apply during the steam generator replacement outage (2R14).

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.
- b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

---

1. All nonplugged tubes that previously had detectable wall penetrations (>20%).
  2. Tubes in those areas where experience has indicated potential problems.
  3. A tube inspection (pursuant to Specification 4.4.5.4.a.9) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.
- c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial inspection provided:
1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.
  2. The inspections include those portions of the tubes where imperfections were previously found.

The result of each sample inspection shall be classified into one to the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.