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JOSEPH A. WIDAY  
VICE PRESIDENT & PLANT MANAGER  
GINNA STATION

December 20, 2000

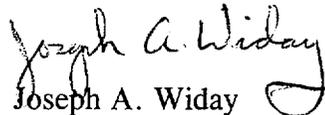
U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Guy S. Vissing  
Project Directorate I  
Washington, D.C. 20555

Subject: Emergency Operating Procedures  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

  
Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

ATT Index  
E Index  
ATT-11.2, Rev. 1  
E-2, Rev. 9

AD02

PARAMETERS: DOC TYPES - PRAR PRATT PRE STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ATT-1.0	ATTACHMENT AT POWER CCW ALIGNMENT	001	07/26/94	02/10/98	02/10/03	EF
ATT-1.1	ATTACHMENT NORMAL CCW FLOW	000	05/18/00	05/18/00	05/18/05	EF
ATT-2.1	ATTACHMENT MIN SW	004	06/26/98	02/10/98	02/10/03	EF
ATT-2.2	ATTACHMENT SW ISOLATION	006	03/25/99	08/11/98	08/11/03	EF
ATT-2.3	ATTACHMENT SW LOADS IN CNMT	003	01/25/95	12/31/99	12/31/04	EF
ATT-3.0	ATTACHMENT CI/CVI	005	01/25/99	01/06/99	01/06/04	EF
ATT-3.1	ATTACHMENT CNMT CLOSURE	003	01/25/99	01/25/99	01/25/04	EF
ATT-4.0	ATTACHMENT CNMT RECIRC FANS	003	07/26/94	05/13/98	05/13/03	EF
ATT-5.0	ATTACHMENT COND TO S/G	004	01/25/95	12/31/99	12/31/04	EF
ATT-5.1	ATTACHMENT SAFW	006	07/07/98	12/31/99	12/31/04	EF
ATT-5.2	ATTACHMENT FIRE WATER COOLING TO TDAFW PUMP	003	01/14/99	01/14/99	01/14/04	EF
ATT-6.0	ATTACHMENT COND VACUUM	003	12/18/96	02/10/98	02/10/03	EF
ATT-7.0	ATTACHMENT CR EVAC	005	02/11/00	02/10/98	02/10/03	EF
ATT-8.0	ATTACHMENT DC LOADS	006	03/22/99	01/14/99	01/14/04	EF
ATT-8.1	ATTACHMENT D/G STOP	004	11/03/95	02/10/98	02/10/03	EF
ATT-8.2	ATTACHMENT GEN DEGAS	006	08/17/99	08/17/99	08/17/04	EF
ATT-8.3	ATTACHMENT NONVITAL	003	07/26/94	02/10/98	02/10/03	EF
ATT-8.4	ATTACHMENT SI/UV	004	04/24/97	02/10/98	02/10/03	EF
ATT-9.0	ATTACHMENT LETDOWN	007	06/09/00	01/06/99	01/06/04	EF
ATT-9.1	ATTACHMENT EXCESS L/D	003	03/31/00	02/10/98	02/10/03	EF
ATT-10.0	ATTACHMENT FAULTED S/G	005	10/03/96	05/13/98	05/13/03	EF
ATT-11.0	ATTACHMENT IA CONCERNS	002	04/07/97	08/11/98	08/11/03	EF
ATT-11.1	ATTACHMENT IA SUPPLY	002	04/07/97	08/11/98	08/11/03	EF
ATT-11.2	ATTACHMENT DIESEL AIR COMPRESSOR	001	12/20/00	04/03/98	04/03/03	EF

PARAMETERS: DOC TYPES - PRAR PRATT PRE STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ATT-12.0	ATTACHMENT N2 PORVS	003	03/24/97	02/10/98	02/10/03	EF
ATT-13.0	ATTACHMENT NC	002	07/26/94	02/10/98	02/10/03	EF
ATT-14.0	ATTACHMENT NORMAL RHR COOLING	002	04/07/97	09/23/99	09/23/04	EF
ATT-14.1	ATTACHMENT RHR COOL	004	05/01/98	05/01/98	05/01/03	EF
ATT-14.2	ATTACHMENT RHR ISOL	001	07/26/94	02/10/98	02/10/03	EF
ATT-14.3	ATTACHMENT RHR NPSH	002	08/01/97	01/06/99	01/06/04	EF
ATT-14.4	ATTACHMENT RHR SAMPLE	001	07/26/94	01/06/99	01/06/04	EF
ATT-14.5	ATTACHMENT RHR SYSTEM	002	07/26/94	02/10/98	02/10/03	EF
ATT-14.6	ATTACHMENT RHR PRESS REDUCTION	001	01/14/99	01/14/99	01/14/04	EF
ATT-15.0	ATTACHMENT RCP START	006	10/13/00	03/17/00	03/17/05	EF
ATT-15.1	ATTACHMENT RCP DIAGNOSTICS	003	04/24/97	02/10/98	02/10/03	EF
ATT-15.2	ATTACHMENT SEAL COOLING	003	05/22/97	02/10/98	02/10/03	EF
ATT-16.0	ATTACHMENT RUPTURED S/G	009	01/11/00	01/11/00	01/11/05	EF
ATT-16.1	ATTACHMENT SGTI	000	09/08/00	09/08/00	09/08/05	EF
ATT-16.2	ATTACHMENT RCS BORON FOR SGTI	001	10/13/00	09/08/00	09/08/05	EF
ATT-17.0	ATTACHMENT SD-1	010	10/13/00	02/29/00	02/28/05	EF
ATT-17.1	ATTACHMENT SD-2	005	09/26/96	09/10/96	09/10/01	EF
ATT-18.0	ATTACHMENT SFP - RWST	004	10/08/97	02/10/98	02/10/03	EF
ATT-20.0	ATTACHMENT VENT TIME	003	07/26/94	02/10/98	02/10/03	EF
ATT-21.0	ATTACHMENT RCS ISOLATION	001	07/26/94	02/10/98	02/10/03	EF
ATT-22.0	ATTACHMENT RESTORING FEED FLOW	001	02/12/99	03/24/97	03/24/02	EF
ATT-23.0	ATTACHMENT TRANSFER 4160V LOADS	000	02/26/99	02/26/99	02/26/04	EF
ATT-24.0	ATTACHMENT TRANSFER BATTERY TO TSC	000	09/08/00	09/08/00	09/08/05	EF

TOTAL FOR PRATT 47

REPORT NO. 01  
REPORT: NPSP0200  
DOC TYPE: PRE

GINNA NUCLEAR POWER PLANT  
PROCEDURES INDEX  
EMERGENCY PROCEDURE

12/20/00 PAGE: 32

PARAMETERS: DOC TYPES - PRAR PRATT PRE

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
E-0	REACTOR TRIP OR SAFETY INJECTION	027	05/18/00	05/01/98	05/01/03	EF
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	019	12/02/99	05/01/98	05/01/03	EF
E-2	FAULTED STEAM GENERATOR ISOLATION	009	12/20/00	05/01/98	05/01/03	EF
E-3	STEAM GENERATOR TUBE RUPTURE	026	03/31/00	05/01/98	05/01/03	EF
TOTAL FOR PRE	4					

EOP: ATT-11.2	TITLE: ATTACHMENT DIESEL AIR COMPRESSOR	REV: 1 PAGE 1 of 2
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Responsible Manager     *Bob Delinger*     Date     12-20-2000    

**CAUTION: THE PORTABLE DIESEL DRIVEN AIR COMPRESSOR SHOULD ONLY BE USED TO SUPPLY INSTRUMENT AIR IN AN EMERGENCY SO AS TO PREVENT OIL CONTAMINATION OF THE INSTRUMENT AIR SYSTEM.**

To supply Instrument Air System using the Diesel Air Compressor perform the following:

1. Verify hose installed between Diesel Compressor discharge valve (located on compressor) and service air connection (at V-7203C) by Turbine Building overhead door #10.
2. Start the portable Diesel Driven Air Compressor using instructions mounted locally on the compressor.

**NOTE: Blowdown portable Diesel Air Compressor Discharge Filter hourly after initial system start until oil and water are sufficiently removed from header, THEN blowdown the Discharge Filter once every 4 hours.**

3. Blowdown portable Diesel Air Compressor Discharge Filter by cycling open and closed Service Air Yard inlet drain/trap drain valve V-7203B until only air escapes. (Next to Door #10, Column A-7 Turbine Building Basement North)
4. Check closed Breathing Air Compressor discharge isolation valve V-14007Y (next to Door #10, Column A-7 Turbine Building Basement North).
5. Check closed Backup Air Compressor discharge drain valve V-14007X (next to Door #10, Column A-7 Turbine Building Basement North).
6. Open Service Air isolation valve to Turbine Building from Yard, V-7203. (Next to Door #10, Column A-7 Turbine Building Basement North)
7. Blowdown residue in line by throttling open Backup Air Compressor discharge drain valve V-14007X until residue is removed, THEN close drain valve V-14007X.
8. Open Back-up Air Compressors discharge inner isolation valve to Service Air header, V-7195A. (Next to Door #10, Column A-7 Turbine Building Basement Northwest).
9. Check open Back-up Air Compressors discharge outer isolation valve to Service Air header, V-7195. (Overhead by Door #10 Column A-7 Turbine Building Basement Northwest)
10. Open Service Air crosstie valve to Instrument Air System V-5365 to crosstie the Service Air and Instrument Air Systems. (Bypasses AOV-5251, just West of Instrument Air Receivers)

EOP: ATT-11.2	TITLE: ATTACHMENT DIESEL AIR COMPRESSOR	REV: 1 PAGE 2 of 2
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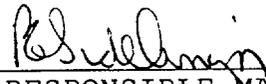
11. IF Instrument Air header pressure is greater than normal operating pressure (~ 120 psig) when using the Gardner Denver Diesel Driven Air Compressor, THEN throttle open V-GD1B, Compressor discharge service isolation valve to maintain Instrument Air header at ~ 120 psig.
12. Maintain proper fuel oil level in portable Diesel Driven Air Compressor. (Notify Maintenance if necessary)
13. Inform I&C that Instrument Air and Service Air are crosstied.

EOP: E-2	TITLE: FAULTED STEAM GENERATOR ISOLATION	REV: 9 PAGE 1 of 8
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ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

  
\_\_\_\_\_

RESPONSIBLE MANAGER

12-20-2000

EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: \_\_\_\_\_

EOP: E-2	TITLE: FAULTED STEAM GENERATOR ISOLATION	REV: 9 PAGE 2 of 8
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A. PURPOSE - This procedure provides actions to identify and isolate a faulted steam generator.

B. ENTRY CONDITIONS/SYMPTOMS

1. ENTRY CONDITIONS - This procedure is entered from:

a. E-0, REACTOR TRIP OR SAFETY INJECTION, with the following symptoms:

- 1) Any S/G pressure decreasing in an uncontrolled manner.
- 2) Any S/G completely depressurized.

b. E-1, LOSS OF REACTOR OR SECONDARY COOLANT, E-3, STEAM GENERATOR TUBE RUPTURE, ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED, and ECA-3.2, SGTR WITH LOSS OF REACTOR COOLANT - SATURATED RECOVERY DESIRED, with the following symptoms and/or conditions:

- 1) Any S/G pressure decreasing in an uncontrolled manner.
- 2) Any S/G completely depressurized.
- 3) Faulted S/G isolation not verified.

c. FR-H.5, RESPONSE TO STEAM GENERATOR LOW LEVEL, when the affected S/G is identified as faulted.

d. Other procedures whenever a faulted S/G is identified.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

\*\*\*\*\*

CAUTION

- o AT LEAST ONE S/G SHALL BE MAINTAINED AVAILABLE FOR RCS COOLDOWN.
- o ANY FAULTED S/G OR SECONDARY BREAK SHOULD REMAIN ISOLATED DURING SUBSEQUENT RECOVERY ACTIONS UNLESS NEEDED FOR RCS COOLDOWN.

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- NOTE:
- o Critical Safety Function Status Trees should be monitored.
  - o Adverse CNMT values should be used whenever CNMT pressure is greater than 4 psig or CNMT radiation is greater than  $10^{+05}$  R/hr.

1 Check MSIV Of Faulted S/G(s)  
- CLOSED

Manually close valve.

IF valve will NOT close from MCB,  
THEN dispatch AO with locked valve  
key to locally closed faulted  
S/G(s) MSIV as follows:

- o S/G A
  - close IA to MSIV, V-5408A
  - open vent valves V-5471 AND  
V-5473
- o S/G B
  - close IA to MSIV, V-5409B
  - open vent valves V-5472 AND  
V-5474

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2	<p>Check If Any S/G Secondary Side Is Intact:</p> <ul style="list-style-type: none"> <li>o Check pressure in S/G A - STABLE OR INCREASING</li> <li style="text-align: center;">-OR-</li> <li>o Check pressure in S/G B - STABLE OR INCREASING</li> </ul>	<p><u>IF</u> both S/G pressures decreasing in an uncontrolled manner, <u>THEN</u> go to ECA-2.1, UNCONTROLLED DEPRESSURIZATION OF BOTH STEAM GENERATORS, Step 1.</p>
3	<p>Check Faulted S/G Status:</p> <ul style="list-style-type: none"> <li>o Faulted S/G pressure - DECREASING IN AN UNCONTROLLED MANNER</li> <li style="text-align: center;">-OR-</li> <li>o Faulted S/G - COMPLETELY DEPRESSURIZED</li> </ul>	<p><u>IF</u> both S/G pressures stable or increasing, <u>THEN</u> search for initiating break and go to Step 6.</p> <ul style="list-style-type: none"> <li>• Main steamlines</li> <li>• Main feedlines</li> <li>• S/G blowdown system</li> <li>• Sample system</li> </ul>

## STEP

## ACTION/EXPECTED RESPONSE

## RESPONSE NOT OBTAINED

4 Isolate Feed Flow To Faulted  
- S/G:

- o Close faulted S/G MDAFW pump discharge valve
  - S/G A, MOV-4007
  - S/G B, MOV-4008
- o Pull stop faulted S/G MDAFW pump
- o Close faulted S/G TDAFW flow control valve
  - S/G A, AOV-4297
  - S/G B, AOV-4298
- o Verify faulted S/G MFW regulating valve and bypass valve - CLOSED
  - S/G A, HCV-466 and HCV-480
  - S/G B, HCV-476 and HCV-481
- o Verify MDAFW pump crosstie valves - BOTH CLOSED
  - MOV-4000A
  - MOV-4000B
- o Close faulted S/G SAFW pump discharge valve
  - S/G A, MOV-9701A
  - S/G B, MOV-9701B

Manually close valves.

IF valves can NOT be closed, THEN dispatch AO to locally isolate flowpaths as necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

\*\*\*\*\*

CAUTION.

IF THE TDAFW PUMP IS THE ONLY AVAILABLE SOURCE OF FEED FLOW, THEN STEAM SUPPLY TO THE TDAFW PUMP MUST BE MAINTAINED FROM ONE S/G.

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- 5 Isolate Steam Flow From Faulted S/G:
- o Verify faulted S/G ARV - CLOSED
    - S/G A, AOV-3411
    - S/G B, AOV-3410
  - o Close faulted S/G TDAFW pump steam supply valve and place in PULL STOP
    - S/G A, MOV-3505A
    - S/G B, MOV-3504A
  - o Verify faulted S/G blowdown and sample valves - CLOSED
    - S/G A, AOV-5738 and AOV-5735
    - S/G B, AOV-5737 and AOV-5736
  - o Dispatch AO to complete faulted S/G isolation (Refer to Attachment FAULTED S/G)

Manually close valves.  
  
IF valves can NOT be closed, THEN dispatch AO to locally isolate flowpaths as necessary.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

\*\*\*\*\*

CAUTION

IF CST LEVEL DECREASES TO LESS THAN 5 FEET, THEN ALTERNATE WATER SOURCES FOR AFW PUMPS WILL BE NECESSARY (REFER TO ER-AFW.1, ALTERNATE WATER SUPPLY TO AFW PUMPS).

\*\*\*\*\*

NOTE: TDAFW pump flow control valves fail open on loss of IA.

\* 6 Monitor Intact S/G Levels:

a. Narrow range level - GREATER THAN 5% [25% adverse CNMT]

a. Maintain total feed flow greater than 200 gpm until narrow range level greater than 5% [25% adverse CNMT] in at least one S/G.

b. Control feed flow to maintain narrow range level between 17% [25% adverse CNMT] and 50%

b. IF narrow range level in any S/G continues to increase in an uncontrolled manner, THEN go to E-3, STEAM GENERATOR TUBE RUPTURE, Step 1.

7 Check Secondary Radiation Levels - NORMAL

o Steamline radiation monitor (R-31 and R-32)

IF steamline radiation monitors NOT available, THEN dispatch AO to locally check steamline radiation.

o Air ejector radiation monitor (R-15)

IF abnormal radiation levels detected in any S/G, THEN go to E-3, STEAM GENERATOR TUBE RUPTURE, Step 1.

o S/G blowdown radiation monitor (R-19)

o Request RP sample S/Gs for activity

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

8 Adjust Steam Dump To Minimize  
RCS Heatup:

- a. Determine allowable intact S/G pressure using maximum hot leg temperature (Refer to Figure INTACT S/G PRESSURE)
  - b. Check condenser steam dump available:
    - o Verify intact S/G MSIV - OPEN
    - o Annunciator G-15, STEAM DUMP ARMED- LIT
  - c. Verify steam dump mode selector switch in MANUAL
  - d. Adjust condenser steam dump controller in AUTO to pressure determined from Figure INTACT S/G PRESSURE
- b. Perform the following:
    - 1) Adjust intact S/G ARV to pressure determined from Figure INTACT S/G PRESSURE.
    - 2) Go to E-1, LOSS OF REACTOR OR SECONDARY COOLANT, Step 1.

9 Go To E-1, LOSS OF REACTOR OR  
SECONDARY COOLANT, Step 1

- END -

EOP:  E-2	TITLE:  FAULTED STEAM GENERATOR ISOLATION	REV: 9  PAGE 1 of 1
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E-2 APPENDIX LIST

TITLE

- 1) FIGURE INTACT S/G PRESSURE (FIG-7.0)
- 2) ATTACHMENT FAULTED S/G (ATT-10.0)