

Mr. Robert G. Byram
 Senior Vice President-Near
 Pennsylvania Power and Light Company
 2 North Ninth Street
 Allentown, PA 18101

December 8, 1997

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M97167 AND M97168)

Dear Mr. Byram:

The Commission has issued the enclosed Amendment No. 170 to Facility Operating License No. NPF-14 and Amendment No. 143 to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station (SSES), Units 1 and 2. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 7, 1996, as supplemented on May 9, 1997.

These amendments modify the SSES, Units 1 and 2 TSs by revising the trip setpoints and allowable values for the secondary containment isolation "Refuel Floor High Exhaust Duct Radiation-High" monitor, the "Railroad Access Shaft Exhaust Duct Radiation-High" monitor, and the "Refuel Floor Wall Exhaust Duct Radiation-High" monitor, in Table 3.3.2-2.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

/s/
 Chester Poslusny, Senior Project Manager
 Project Directorate I-2
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Docket Nos. 50-387/388

- Enclosures: 1. Amendment No. 170 to License No. NPF-14
 2. Amendment No. 143 to License No. NPF-22
 3. Safety Evaluation

DF01/1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 8, 1997

Mr. Robert G. Byram
Senior Vice President-Nuclear
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, PA 18101

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M97167
AND M97168)

Dear Mr. Byram:

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These amendments modify the SSES, Units 1 and 2 TSs by revising the trip setpoints and allowable values for the secondary containment isolation "Refuel Floor High Exhaust Duct Radiation-High" monitor, the "Railroad Access Shaft Exhaust Duct Radiation-High" monitor, and the "Refuel Floor Wall Exhaust Duct Radiation-High" monitor, in Table 3.3.2-2.

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Chester Poslusny, Senior Project Manager
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Division of Reactor Projects - I/II
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3. Safety Evaluation

cc w/encls: See next page

Mr. Robert G. Byram
Pennsylvania Power & Light Company

Susquehanna Steam Electric Station,
Units 1 & 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated October 7, 1996, as supplemented on May 9, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

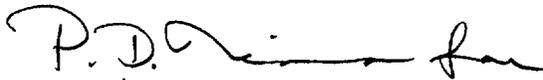
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 170 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and is to be implemented within 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 8, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 170

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 3-17

INSERT

3/4 3-17

TABLE 3.3.2-2 ISOLATION ACTUATION INSTRUMENTATION SETPOINTS		
TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level		
1) Low, Level 3	≥ 13.0 inches*	≥ 11.5 inches
2) Low Low, Level 2	≥ -38.0 inches*	≥ -45.0 inches
3) Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure-High	≤ 1.72 psig	≤ 1.88 psig
c. Manual Initiation	NA	NA
d. SGTS Exhaust Radiation-High	≤ 23.0 mR/hr	≤ 31.0 mR/hr
e. Main Steam Line Radiation-High	≤ 7.0 x full power background	≤ 8.4 x full power background
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level-Low Low, Level 2	≥ -38.0 inches*	≥ -45.0 inches
b. Drywell Pressure-High	≤ 1.72 psig	≤ 1.88 psig
c. Refuel Floor High Exhaust Duct Radiation-High	≤ 18 mR/hr	≤ 25 mR/hr
d. Railroad Access Shaft Exhaust Duct Radiation-High	≤ 5 mR/hr	≤ 7 mR/hr
e. Refuel Floor Wall Exhaust Duct Radiation-High	≤ 21 mR/hr	≤ 28 mR/hr
f. Manual Initiation	NA	NA
3. MAIN STEAM LINE ISOLATION		
a. Reactor Vessel Water Level-Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Main Steam Line Radiation-High	≤ 7.0 x full power background	≤ 8.4 x full power background
c. Main Steam Line Pressure-Low	≥ 861 psig	>841 psig
d. Main Steam Line Flow-High	≤ 113 psid**	<121 psid**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-388

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143
License No. NPF-22

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated October 7, 1996, as supplemented on May 9, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

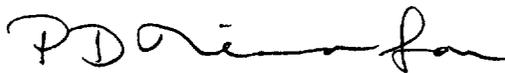
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 143 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and is to be implemented within 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 8, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 143

FACILITY OPERATING LICENSE NO. NPF-22

DOCKET NO. 50-388

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 3-17

INSERT

3/4 3-17

TABLE 3.3.2-2		
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS		
TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
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c. Main Steam Line Pressure-Low	≥ 861 psig	≥ 841 psig
d. Main Steam Line Flow-High	≤ 113 psid**	≤ 121 psid**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. NPF-14
AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. NPF-22
PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2
DOCKET NOS. 50-387 AND 388

1.0 INTRODUCTION

By letter dated October 7, 1996, and as supplemented by letter dated May 9, 1997, the Pennsylvania Power and Light Company (PP&L, the licensee) submitted a proposed amendment to the Susquehanna Steam Electric Station (SSES), Units 1 and 2, Technical Specifications (TSs) Table 3.3.2-2. The proposed amendment would increase the setpoints and allowable values for the refuel floor high exhaust duct radiation - high monitor, the railroad access shaft exhaust duct radiation - high monitor, and the refuel floor wall exhaust duct radiation - high monitor. These monitors are part of the secondary containment isolation actuation instrumentation. By increasing the trip setpoints and the allowable values, activities that cause spurious actuation of the secondary containment isolation system will be reduced during operation of the hydrogen water chemistry (HWC) process. As a result of the licensee's plan to implement the HWC process to enhance protection of the lower reactor vessel internals from the effects of intergranular stress corrosion cracking, background radiation exposure rates would increase, which for the refuel floor wall exhaust duct radiation - high monitors, are expected to exceed the monitor's existing trip setpoint. This proposed modification would increase the trip setpoints and allowable values to the secondary containment isolation radiation monitors to allow for the operation of the HWC process. The May 9, 1997, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The SSES Units 1 and 2 TS Table 3.2.2-2 currently contains secondary containment radiation monitor isolation trip setpoints and allowable values for the refuel floor high exhaust duct radiation - high monitor, the railroad access shaft exhaust duct radiation - high monitor, and the refuel floor wall exhaust duct radiation - high monitor. These monitors are process radiation monitors and are part of the secondary containment isolation actuation instrumentation. These process radiation monitors are designed to isolate Zone III of the secondary containment, initiate the standby gas treatment

system (SGTS), and initiate the reactor building recirculation system on high radiation resulting from fuel handling accidents. Therefore, the design basis for these process radiation monitors is to monitor radioactive materials in the unfiltered air from the Zone III exhaust system, and provide signals to limit offsite doses to maintain regulatory requirements. Zone III of the secondary containment includes the refueling floor and can include the railroad access shaft during certain alignments. The primary function of these process radiation monitors is the isolation or operation of certain ventilation systems, and the secondary function is to provide radiation protection to workers. Therefore, an increase to the trip setpoints and allowable values will not result in a decrease of the safety function of the monitors.

The refuel floor wall exhaust duct radiation monitoring system (monitors RE-1N010A & RE-1N010B) monitors the radiation level in the exhaust duct from the refueling floor prior to its discharge to the atmosphere through the reactor building vent. The detection assemblies are located in the exhaust ducting upstream of the inboard isolation damper. The alarms, consisting of two alarms for high radiation (high and high-high) and one alarm for downscale trip, are located in the control room. Two trip circuits monitor the upscale (high-high)/inoperative condition and the downscale condition. The upscale trip indicates high radiation levels. It is this upscale trip which initiates closure of the reactor building Zone III ventilation outboard isolation dampers, starts the SGTS and the reactor building recirculation system.

The refueling floor exhaust duct high radiation monitoring system (monitors RE-1N015A & RE-1N015B) consists of shielded detectors whose purpose is to monitor the radiation level in the refueling floor ventilation exhaust duct adjacent to the intake register. The system is identical to the refueling floor wall exhaust radiation monitoring system with trip logic and protective action initiation.

The railroad access shaft exhaust duct radiation monitoring system (monitors RE-1N016A & RE-1N016B) monitors the radiation level at the air exhaust duct prior to the reactor building vent. The system design is identical to the refueling wall radiation monitoring system with trip logic and protective action initiation.

PP&L is planning to utilize the HWC process in the SSES units to enhance protection of the lower vessel internals from the effects of intergranular stress corrosion cracking (IGSCC). Operation of the HWC process results in increased background radiation exposure rates which, for the refuel floor wall exhaust duct radiation - high monitor, are expected to exceed the monitor's existing trip setpoint. To allow for plant operation with the HWC process, and to reduce activities that unnecessarily challenge plant safety systems (spurious actuations), the trip setpoints and allowable values must be increased.

The original setpoints for the above monitors were conservatively based upon normal operating conditions, instead of accident conditions, and were set at a value to preclude spurious design actuation of these monitors during normal plant operations. However, as required by the TSs the monitors are OPERABLE when conditions exist that may result in fuel damage events. The revised limits were derived reflecting the accident function of the monitors, using FSAR realistic source terms for refueling accidents and calculating exposure dose rates at the location of the monitors. As indicated in the licensee's calculation (EC-RADN-0531) when calculating setpoints, using realistic source terms results in lower radioactivity on the refueling floor and a lower setpoint when compared to using design basis source terms. The release of FSAR realistic source term, without mitigation (that is no secondary containment isolation or activation of SGTS), would produce lower offsite doses than the design basis source term with mitigation. Therefore, it is more conservative to determine setpoints by the FSAR realistic source term than by the design basis source term. These calculated setpoints were then adjusted downward to account for instrument and calibration accuracies and instrument drift tolerances and listed in TS Table 3.3.2-2. The instrument and calibration accuracies were calculated based on GE standard setpoint change methodologies and were listed as the allowable values in TS Table 3.3.2-2. These setpoints will assure that secondary containment is isolated and SGTS is initiated when necessary to protect public health and safety and will also be high enough to permit plant operation and to reduce the potential for spurious activation. The offsite dose consequences will remain below the calculated design basis dose consequences even without credit for SGTS system function while using the realistic source term to determine the monitor setpoint. The dose consequences associated with the design basis fuel handling accident (as stated in FSAR Section 15.7.4) continues to be the licensing basis commitment. The review of the dose consequences is discussed below.

The proposed TS change would increase the trip setpoints and allowable values for the above mentioned process radiation monitors in Table 3.3.2-2. This revision is necessary because the background radiation fields at the refueling floor wall exhaust and high exhaust monitors are expected to approach or to exceed current trip and alarm setpoints under moderate operation of the HWC process. The following are the proposed trip setpoints and allowable values for the process radiation monitors in question:

(proposed) TECHNICAL SPECIFICATIONS TABLE 3.3.2-2 ISOLATION ACTUATION INSTRUMENTATION SETPOINTS		
Secondary Containment Isolation		
Trip Function	Trip Setpoint	Allowable Value
c. Refuel Floor High Exhaust Duct Radiation - High (RE-1N015A,B)	18 mR/hr	25 mR/hr
d. Railroad Access Shaft Exhaust Duct Radiation - High (RE-1N016A,B)	5 mR/hr	7 mR/hr
e. Refuel Floor Wall Exhaust Duct Radiation - High (RE-1N010A,B)	21 mR/hr	28 mR/hr

The staff reviewed a radiation dose analysis performed by the licensee. The radiation dose analysis was performed to assess the radiological impacts of the HWC process on the process and area radiation monitors at SSES. The calculated N-16 dose rates increased by a factor of 5 for the refueling floor wall exhaust duct radiation - high monitors (from 1.61 mrad/hour to 8.04 mrad/hour) and the refueling floor high exhaust duct radiation - high monitors (from 0.085 mrad/hour to 0.43 mrad/hour). The results for the Unit 1 analysis are as follows (Unit 2 is expected to be similar):

Monitor	Current Background	Calculated N-16 Dose Rates		Current T.S. Setpoints	
	A/B (mR/hr)	Without HWC (mR/hr)	With HWC (mR/hr)	Alarm (mR/hr)	Trip (mR/hr)
Refueling Floor Wall Exhaust (RE-1N010A,B)	0.339/0.501	1.61	8.04	≤1.0	≤2.5
Refueling Floor High Exhaust (RE-1N015A,B)	0.054/0.063	0.085	0.43	≤1.0	≤2.5
Railroad Access Shaft Exhaust (RE-1N016A,B)	0.170/0.355	<0.01	<0.01	≤1.0	≤2.5

For comparison, the current trip setpoints for the main steam line radiation monitors have been analyzed. These trip setpoints are 7 times the normal radiation background at 100% power, with an allowable value of 8.4 times the normal radiation background. The alarm setpoint is 3.5 times the radiation background level. Since the operation of the HWC process is expected to increase main steam line radiation levels to approximately 5 times normal radiation background values, the alarm setpoint will be exceeded, but the trip and allowable values will not, under steady-state conditions.

With the operation of the HWC process, however, a plant chemistry transient could increase N-16 volatility further, up to 8.5 times the normal radiation background levels. With the current trip setpoint, this would result in closure of the main steam isolation valves and actuation of the reactor protection system, producing an unnecessary unit scram. Revised trip and alarm setpoints for the main steam line radiation monitors are expected to be submitted to the staff for review in the near future. These setpoints are expected to be sufficiently high to avoid unnecessary plant trips, but low enough to detect a major fuel failure.

The staff has evaluated PP&L's proposed amendment to the SSES TSs. In its evaluation, the staff considered the licensee's submittal and the increase in the background radiation exposure rates that would accompany the operation of the HWC process. The staff has concluded that although the increase in background radiation exposure rates will raise the detection threshold of certain monitors, adequate sensitivity continues to exist to detect off-normal releases whose offsite impact is small relative to 10 CFR Part 100 acceptance criteria. Therefore, the licensee's proposal to increase the TSs trip setpoints and allowable values to the proposed values, for the "refuel floor high exhaust duct radiation - high", the "railroad access shaft exhaust duct radiation - high" and the "refuel floor wall exhaust duct radiation - high" secondary containment radiation monitors, is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 66716). Accordingly, the amendments

meet eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: N. Stinson
C. Li

Date: December 8, 1997