



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-8064

January 11, 2001

Name of Licensee: Arizona Public Service Company  
Energy Northwest  
Entergy Operations, Inc.  
Nebraska Public Power District  
Omaha Public Power District  
Pacific Gas and Electric Company  
STP Nuclear Operating Company  
Southern California Edison Company  
TXU Electric  
Union Electric Company  
Wolf Creek Nuclear Operating Corporation

SUBJECT: MEETING SUMMARY OF THE NOVEMBER 15, 2000, PUBLIC WORKSHOP -  
REGION IV

Dear Sirs:

This letter provides a meeting summary for the NRC sponsored Public Workshop conducted in Arlington, Texas, on November 15, 2000, at the Hilton Arlington Hotel. The workshop was open to public participation and involved discussions between Region IV utility representatives, the public, and the NRC staff.

Enclosed is the workshop summary, including a list of attendees. We found the workshop to be a valuable opportunity for the exchange of ideas and concerns between the NRC staff and Region IV licensee representatives concerning the initial implementation of the reactor oversight process.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ken E. Brockman, Director  
Division of Reactor Projects

Dockets:

50-313; 50-368  
50-483  
50-445; 50-446  
50-298  
50-275; 50-323  
50-285  
50-416  
50-528; 50-529; 50-530  
50-458  
50-361; 50-362  
50-498; 50-499  
50-382  
50-397  
50-482

Enclosures:

1. November 15, 2000, Public Workshop Summary
2. List of Public Meeting Attendees

cc w/enclosures:

Craig G. Anderson, Vice President,  
Operations  
Arkansas Nuclear One  
Entergy Operations, Inc.  
1448 S.R. 333  
Russellville, Arkansas 72801-0967

Executive Vice President  
& Chief Operating Officer  
Entergy Operations, Inc.  
P.O. Box 31995  
Jackson, Mississippi 39286-1995

Vice President  
Operations Support  
Entergy Operations, Inc.  
P.O. Box 31995  
Jackson, Mississippi 39286-1995

Manager, Washington Nuclear Operations  
ABB Combustion Engineering Nuclear  
Power  
12300 Twinbrook Parkway, Suite 330  
Rockville, Maryland 20852

County Judge of Pope County  
Pope County Courthouse  
100 West Main Street  
Russellville, Arkansas 72801

Winston & Strawn  
1400 L Street, N.W.  
Washington, DC 20005-3502

David D. Snellings, Jr., Director  
Division of Radiation Control and  
Emergency Management  
Arkansas Department of Health  
4815 West Markham Street, Mail Slot 30  
Little Rock, Arkansas 72205-3867

Manager  
Rockville Nuclear Licensing  
Framatome Technologies  
1700 Rockville Pike, Suite 525  
Rockville, Maryland 20852

OES Coordinator  
Pope County  
No. 4 Emergency Lane

Russellville, Arkansas 72801

ES Coordinator  
Yell County Courthouse  
Union Street  
Dardenelle, Arkansas 72834

ES Coordinator  
Johnson County OES  
P.O. Box 546  
Clarksville, Arkansas 72830

ES Coordinator  
Logan County OES  
Logan County Courthouse  
Paris, Arkansas 72855

ES Coordinator  
Conway County  
105 Eisenhower Street  
Morrilton, Arkansas 72110

Mayor, City of Danville  
P.O. Box 69  
Danville, Arkansas 72833

Mayor, City of Paris  
P.O. Box 271  
Paris, Arkansas 72855

Mayor, City of Clarksville  
205 Walnut Street  
Clarksville, Arkansas 72830-3005

Mayor, City of Dardanelle  
P.O. Box 360  
Dardanelle, Arkansas 72834

Mayor, City of Dover  
P.O. Box 258  
Dover, Arkansas 72837

Mayor, City of Atkins  
P.O. Box 128  
Atkins, Arkansas 72823

Mayor, City of Russellville

P.O. Box 428  
Russellville, Arkansas 72811

Mayor, City of London  
P.O. Box 130  
London, Arkansas 72847

Mayor, City of Morrilton  
City Hall  
P.O. Box 583  
Morrilton, Arkansas 72110

Judge, Yell County  
Danville Courthouse  
P.O. Box 219  
Danville, Arkansas 72833

County Judge  
Conway County Courthouse  
117 South Moose, Room 203  
Morrilton, Arkansas 72110

County Judge  
Johnson County Courthouse  
P.O. Box 278  
Clarksville, Arkansas 72830

County Judge  
Logan County Courthouse  
Room 22  
Paris, Arkansas 72855

Federal Emergency Management Agency  
R. L. "Buddy" Young, Regional Director  
Region VI, Federal Center  
800 North Loop 288  
Denton, Texas 76201-3698

J. H. Swailes, Vice President of  
Nuclear Energy  
Nebraska Public Power District  
P.O. Box 98  
Brownville, Nebraska 68321

G. R. Horn, Senior Vice President  
of Energy Supply  
Nebraska Public Power District

1414 15th Street  
Columbus, Nebraska 68601

John R. McPhail, General Counsel  
Nebraska Public Power District  
P.O. Box 499  
Columbus, Nebraska 68602-0499

S. R. Mahler, Assistant Nuclear  
Licensing and Safety Manager  
Nebraska Public Power District  
P.O. Box 98  
Brownville, Nebraska 68321

Dr. William D. Leech  
Manager - Nuclear  
MidAmerican Energy  
907 Walnut Street  
P.O. Box 657  
Des Moines, Iowa 50303-0657

Ron Stoddard  
Lincoln Electric System  
1040 O Street  
P.O. Box 80869  
Lincoln, Nebraska 68501-0869

Michael J. Linder, Director  
Nebraska Department of Environmental  
Quality  
P.O. Box 98922  
Lincoln, Nebraska 68509-8922

Chairman  
Nemaha County Board of Commissioners  
Nemaha County Courthouse  
1824 N Street  
Auburn, Nebraska 68305

Cheryl K. Rogers, Program Manager  
Nebraska Health and Human Services  
System  
Division of Public Health Assurance  
Consumer Services Section  
301 Centennial Mall, South  
P.O. Box 95007  
Lincoln, Nebraska 68509-5007

Ronald A. Kucera, Director  
of Intergovernmental Cooperation  
Department of Natural Resources  
P.O. Box 176  
Jefferson City, Missouri 65102

Jerry Uhlmann, Director  
State Emergency Management Agency  
P.O. Box 116  
Jefferson City, Missouri 65101

Vick L. Cooper, Chief  
Radiation Control Program, RCP  
Kansas Department of Health  
and Environment  
Bureau of Air and Radiation  
Forbes Field Building 283  
Topeka, Kansas 66620

Chairman  
Nemaha County Board of Commissioners  
Nemaha County Courthouse  
1824 N Street  
Auburn, Nebraska 68305

Presiding Commissioner  
Atchison County  
P.O. Box 280  
Rock Port, Missouri 64482

Sheriff  
Atchison County Sheriff's Office  
511 West Clay  
Rock Port, Missouri 64482

Sheriff  
1805 N Street  
Auburn, Nebraska 68305

John A. Miller, Regional Director  
Federal Emergency Management Agency  
Region VII  
2323 Grand Blvd., Suite 900  
Kansas City, Missouri 64108-2670

Robert Leopold  
State Liaison Officer  
Nebraska Health and Human Services  
Division of Public Health Assurance  
Department of Regulation and Licensure  
P.O. Box 95007  
Lincoln, Nebraska 68509-5007

Daniel K. McGhee  
Bureau of Radiological Health  
Iowa Department of Public Health  
Lucas State Office Building  
321 12th Street  
Des Moines, Iowa 50319-0075

Gary McNutt, Deputy Director,  
Radiation Control  
Section for Environmental Public Health  
P.O. Box 570  
Jefferson City, Missouri 65102-0570

Mr. C. L. Terry  
TXU Electric  
Senior Vice President & Principal Nuclear  
Officer  
ATTN: Regulatory Affairs Department  
P.O. Box 1002  
Glen Rose, Texas 76043

Roger D. Walker  
TXU Electric  
Regulatory Affairs Manager  
P.O. Box 1002  
Glen Rose, Texas 76043

Juanita Ellis  
President - CASE  
1426 South Polk Street  
Dallas, Texas 75224

George L. Edgar, Esq.  
Morgan, Lewis & Bockius  
1800 M. Street, NW  
Washington, D.C. 20036

G. R. Bynog, Program Manager/  
Chief Inspector  
Texas Department of Licensing &  
Regulation  
Boiler Division  
P.O. Box 12157, Capitol Station  
Austin, Texas 78711

County Judge  
P.O. Box 851  
Glen Rose, Texas 76043

Chief, Bureau of Radiation Control  
Texas Department of Health  
1100 West 49th Street  
Austin, Texas 78756-3189

John L. Howard, Director  
Environmental and Natural Resources  
Policy  
Office of the Governor  
P.O. Box 12428  
Austin, Texas 78711-3189

Sheriff, Somervell County  
1505 Northeast Big Bend Trail  
Glen Rose, Texas 76043

Mayor of Glen Rose  
P.O. Box 87  
Glen Rose, Texas 76043

The Honorable Don Cleveland  
County Court Room 7  
Granbury, Texas 76048

Mayor of Granbury  
P.O. Box 969  
Granbury, Texas 76048

Public Utility Commission  
William B. Travis Bldg.  
P.O. Box 13326  
1701 North Congress Avenue  
Austin, Texas 78701-3326

Garry L. Randolph, Vice President and  
Chief Nuclear Officer  
Union Electric Company  
P.O. Box 620  
Fulton, Missouri 65251

Professional Nuclear Consulting, Inc.  
19041 Raines Drive  
Derwood, Maryland 20855

John O'Neill, Esq.  
Shaw, Pittman, Potts & Trowbridge  
2300 N. Street, N.W.  
Washington, D.C. 20037

Mark A. Reidmeyer, Regional  
Regulatory Affairs Supervisor  
Quality Assurance  
Union Electric Company  
P.O. Box 620  
Fulton, Missouri 65251

Manager - Electric Department  
Missouri Public Service Commission  
301 W. High  
P.O. Box 360  
Jefferson City, Missouri 65102

Otto L. Maynard, President and  
Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
P.O. Box 411  
Burlington, Kansas 66839

Dan I. Bolef, President  
Kay Drey, Representative  
Board of Directors Coalition  
for the Environment  
6267 Delmar Boulevard  
University City, Missouri 63130

Lee Fritz, Presiding Commissioner  
Callaway County Courthouse  
10 East Fifth Street  
Fulton, Missouri 65251

Alan C. Passwater, Manager  
Licensing and Fuels  
AmerenUE  
One Ameren Plaza  
1901 Chouteau Avenue  
P.O. Box 66149  
St. Louis, Missouri 63166-6149

J. V. Laux, Manager  
Quality Assurance  
Union Electric Company  
P.O. Box 620  
Fulton, Missouri 65251

Mayor of Chamois  
Star Route  
Chamois, Missouri 65024

Presiding Commissioner  
P.O. Box 826  
Linn, Missouri 65051

Emergency Preparedness  
Coordinator  
P.O. Box 826  
Linn, Missouri 65051

Osage County Sheriff  
P.O. Box 619  
Linn, Missouri 65051

Public Administrator  
Star Route  
Chamois, Missouri 65024

Emergency Preparedness  
Coordinator  
P.O. Box 817  
Fulton, Missouri 65251

Callaway County Sheriff  
P.O. Box 817  
Fulton, Missouri 65251

Presiding Commissioner  
Montgomery County Courthouse  
211 E. Third

Montgomery City, Missouri 63361

Director of Emergency Services  
Montgomery County Courthouse  
211 E. Third  
Montgomery City, Missouri 63361

Montgomery County Sheriff  
Montgomery County Courthouse  
211 E. Third  
Montgomery City, Missouri 63361

Presiding Commissioner  
Gasconade County  
119 E. First Street, Room 2  
Hermann, Missouri 65041

Emergency Preparedness  
Coordinator  
Gasconade County  
119 E. First Street, Room 2  
Hermann, Missouri 65041

Gasconade County Sheriff  
119 E. First Street, Room 22  
Hermann, Missouri 65041

Gasconade County Public  
Administrator  
501 S. Second St.  
Owensville, Missouri 65066

Mayor of Fulton  
P.O. Box 130  
Fulton, Missouri 65251

Chief of Police  
P.O. Box 130  
Fulton, Missouri 65251

Fulton City Administrator  
P.O. Box 130  
Fulton, Missouri 65251

Mayor, City of Morrison  
Route 1  
Morrison, Missouri 65061

Mayor, City of Mokane  
P.O. Box 41  
Mokane, Missouri 65059

Mayor of Rhineland  
P.O. Box 384  
Rhineland, Missouri 65069

Assistant Emergency Director  
Route 1, Box 2  
Rhineland, Missouri 65069

Eastern Associate Commissioner  
10 E. 5th Street  
Fulton, Missouri 65251

Western Associate Commissioner  
10 E. 5th Street  
Fulton, Missouri 65251

Gregory M. Rueger, Senior Vice  
President, Generation and Chief Nuclear  
Officer  
Pacific Gas and Electric Company  
Diablo Canyon Power Plant  
P.O. Box 3  
Avila Beach, CA 93424

David H. Oatley, Vice President  
Diablo Canyon Operations and Plant  
Manager  
Diablo Canyon Nuclear Power Plant  
P.O. Box 56  
Avila Beach, California 93424

Lawrence F. Womack, Vice President,  
Power Generation & Nuclear Services  
Diablo Canyon Power Plant  
P.O. Box 56  
Avila Beach, CA 93434

Dr. Richard Ferguson  
Energy Chair  
Sierra Club California  
1100 11th Street, Suite 311  
Sacramento, California 95814

Nancy Culver  
San Luis Obispo Mothers for Peace  
P.O. Box 164  
Pismo Beach, California 93448

Chairman  
San Luis Obispo County Board of  
Supervisors  
Room 370  
County Government Center  
San Luis Obispo, California 93408

Truman Burns\Mr. Robert Kinoshian  
California Public Utilities Commission  
505 Van Ness, Rm. 4102  
San Francisco, California 94102

Robert R. Wellington, Esq.  
Legal Counsel  
Diablo Canyon Independent Safety  
Committee  
857 Cass Street, Suite D  
Monterey, California 93940

Ed Bailey, Radiation Program Director  
Radiologic Health Branch  
State Department of Health Services  
P.O. Box 942732 (MS 178)  
Sacramento, CA 94327-7320

Steve Hsu  
Radiologic Health Branch  
State Department of Health Services  
P.O. Box 942732  
Sacramento, California 94327-7320

Christopher J. Warner, Esq.  
Pacific Gas and Electric Company  
P.O. Box 7442  
San Francisco, California 94120

City Editor  
The Tribune  
3825 South Higuera Street  
P.O. Box 112  
San Luis Obispo, California 93406-0112

Robert A. Laurie, Commissioner  
California Energy Commission  
1516 Ninth Street (MS 31)  
Sacramento, CA 95814

County Administrative Officer  
San Luis Obispo County Board of  
Supervisors  
Room 370  
County Government Center  
San Luis Obispo, California 93408

S. K. Gambhir, Division Manager  
Nuclear Operations  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023-0399

Mark T. Frans, Manager  
Nuclear Licensing  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023-0399

James W. Chase, Division Manager  
Nuclear Assessments  
Fort Calhoun Station  
P.O. Box 399  
Fort Calhoun, Nebraska 68023

Richard P. Clemens, Manager - Fort  
Calhoun Station  
Omaha Public Power District  
Fort Calhoun Station FC-1-1 Plant  
P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023



Chairman  
Washington County Board of Supervisors  
Washington County Courthouse  
P.O. Box 466  
Blair, Nebraska 68008

Sheriff  
Douglas County  
Hall of Justice  
17th and Farnam  
Omaha, Nebraska 68102

Chairman  
Douglas County Board of Commissioners  
Civic Center  
1819 Farnam  
Omaha, Nebraska 68183

Director  
Douglas County E M  
Civic Center  
EOC, Room 114  
Omaha, Nebraska 68183

Civil Defense Director  
Harrison County  
116 North 2nd Avenue  
Logan, Iowa 51546

Sheriff  
Harrison County  
Sheriff's Department  
Logan, Iowa 51546

Chairman  
Harrison County Board of Supervisors  
Court House  
Logan, Iowa 51546

Mayor, City of Missouri Valley  
City Hall  
223 E. Erie  
Missouri Valley, Iowa 51555

Emergency Management Coordinator  
Pottawattamie County  
227 S. 6th Street, Court House

Council Bluffs, Iowa 51501

Chairman  
Pottawattamie County Board of  
Supervisors  
227 South 6th Street  
Council Bluffs, Iowa 51501

Civil Defense Director  
Washington County Courthouse  
P.O. Box 466  
Blair, Nebraska 68008

Sheriff  
Washington County Courthouse  
1535 Culfax  
Blair, Nebraska 68008

President  
Blair City Council  
218 S. 16th Street  
Blair, Nebraska 68008

Mayor, City of Blair  
218 South 16th Street  
Blair, Nebraska 68008

Mayor, City of Fort Calhoun  
P.O. Box 448  
Fort Calhoun, Nebraska 68023

President  
Fort Calhoun City Council  
P.O. Box 448  
Fort Calhoun, Nebraska 68023

Civil Defense Director  
Region 5/6  
435 N. Park Avenue, Room B101  
Fremont, Nebraska 68025

Chief, Bureau of Energy Efficiency,  
Auditing, and Research  
Utilities Division  
Iowa Department of Commerce  
Lucas State Office Building, 5th Floor  
Des Moines, Iowa 50319

Donald A. Flater, Chief  
Bureau of Environmental Health  
Iowa Department of Public Health  
Lucas State Office Building, 1st Floor  
Des Moines, Iowa 50319

Missouri Public Service Commission  
ATTN: Bruce Deering, Manager -  
Electric Department  
301 W. High  
P.O. Box 360  
Jefferson City, Missouri 65102

Robert Leopold  
State Liaison Officer  
Regulation and Licensure  
Health & Human Services  
P.O. Box 95007  
Lincoln, Nebraska 68509-5007

William A. Eaton, Vice President  
Operations - Grand Gulf Nuclear Station  
Entergy Operations, Inc.  
P.O. Box 756  
Port Gibson, Mississippi 39150

Wise, Carter, Child & Caraway  
P.O. Box 651  
Jackson, Mississippi 39205

Sam Mabry, Director  
Division of Solid Waste Management  
Mississippi Department of Natural  
Resources  
P.O. Box 10385  
Jackson, Mississippi 39209

President, District 1  
Claiborne County Board of Supervisors  
P.O. Box 339  
Port Gibson, Mississippi 39150

General Manager  
Grand Gulf Nuclear Station  
Entergy Operations, Inc.  
P.O. Box 756  
Port Gibson, Mississippi 39150

The Honorable Richard Ieyoub  
Attorney General  
Department of Justice  
State of Louisiana  
P.O. Box 94005  
Baton Rouge, Louisiana 70804-9005

Office of the Governor  
State of Mississippi  
Jackson, Mississippi 39201

Mike Moore, Attorney General  
Frank Spencer, Asst. Attorney General  
State of Mississippi  
P.O. Box 22947  
Jackson, Mississippi 39225

Dr. F. E. Thompson, Jr.  
State Health Officer  
State Board of Health  
P.O. Box 1700  
Jackson, Mississippi 39205

Robert W. Goff, Program Director  
Division of Radiological Health  
Mississippi Dept. of Health  
P.O. Box 1700  
Jackson, Mississippi 39215-1700

Director, Nuclear Safety  
and Regulatory Affairs  
Entergy Operations, Inc.  
P.O. Box 756  
Port Gibson, Mississippi 39150

Vice President, Operations  
Grand Gulf Nuclear Station  
Entergy Operations, Inc.  
P.O. Box 756  
Port Gibson, Mississippi 39150

Supervisor, District 2  
Claiborne County  
202 Magnolia Lane  
Port Gibson, Mississippi 39150

Supervisor, District 5  
Claiborne County  
Route 1, Box 296  
Port Gibson, Mississippi 39150

Vice President, District 4  
Claiborne County Board  
of Supervisors  
P.O. Box 196  
Hermanville, Mississippi 39086

Claiborne County Tax  
Assessor/Collector  
P.O. Box 469  
Port Gibson, Mississippi 39150

Mayor  
Town of Port Gibson  
P.O. Box 466  
Port Gibson, Mississippi 39150

James E. Maher, Director  
Mississippi Emergency  
Management Agency  
P.O. Box 4501, Fondren Station  
Jackson, Mississippi 39296-4501

Mayor  
City of St. Joseph  
P.O. Box 217  
St. Joseph, Louisiana 71366

Edwin Preis, Mayor  
City of Newellton  
P.O. Box 477  
Newellton, Louisiana 71357

Rick Foster, Director  
Tensas Office of Emergency Preparedness  
P.O. Box 768  
St. Joseph, Louisiana 71366

Ronald Wascom, Administrator  
State Liaison Officer  
Department of Environmental Quality  
P.O. Box 82135  
Baton Rouge, Louisiana 70884-2135

Federal Emergency Management  
Agency, Region IV  
ATTN: John B. Copenhagen  
Regional Director  
3003 Chamblee Tucker Road  
Atlanta, Georgia 30341

Gregg R. Overbeck, Senior Vice  
President, Nuclear  
Arizona Public Service Company  
P.O. Box 52034  
Phoenix, Arizona 85072-2034

Steve Olea  
Arizona Corporation Commission  
1200 W. Washington Street  
Phoenix, Arizona 85007

Douglas K. Porter, Senior Counsel  
Southern California Edison Company  
Law Department, Generation Resources  
P.O. Box 800  
Rosemead, California 91770

Chairman  
Maricopa County Board of Supervisors  
301 W. Jefferson, 10th Floor  
Phoenix, Arizona 85003

Aubrey V. Godwin, Director  
Arizona Radiation Regulatory Agency  
4814 South 40 Street  
Phoenix, Arizona 85040

Angela K. Krainik, Director  
Regulatory Affairs  
Arizona Public Service Company  
P.O. Box 52034  
Phoenix, Arizona 85072-2034

John C. Horne, Vice President,  
Power Generation  
El Paso Electric Company  
2702 N. Third Street, Suite 3040  
Phoenix, Arizona 85004

Terry Bassham, Esq.  
General Counsel  
El Paso Electric Company  
123 W. Mills  
El Paso, Texas 79901

John W. Schumann  
Los Angeles Department of Water & Power  
Southern California Public Power Authority  
P.O. Box 51111, Room 1255-C  
Los Angeles, California 90051-0100

David Summers  
Public Service Company of New Mexico  
414 Silver SW, #1206  
Albuquerque, New Mexico 87102

Jarlath Curran  
Southern California Edison Company  
5000 Pacific Coast Hwy. Bldg. DIN  
San Clemente, California 92672

Robert Henry  
Salt River Project  
6504 East Thomas Road  
Scottsdale, Arizona 85251

Jan Kimmell  
Division of Emergency Management  
State of Arizona  
5636 East McDowell Road  
Phoenix, Arizona 85008

Randal K. Edington, Vice President -  
Operations  
River Bend Station  
Entergy Operations, Inc.  
P.O. Box 220  
St. Francisville, Louisiana 70775

General Manager  
Plant Operations  
River Bend Station  
Entergy Operations, Inc.  
P.O. Box 220  
St. Francisville, Louisiana 70775

Director - Nuclear Safety  
River Bend Station  
Entergy Operations, Inc.  
P.O. Box 220  
St. Francisville, Louisiana 70775

Manager - Licensing  
River Bend Station  
Entergy Operations, Inc.  
P.O. Box 220  
St. Francisville, Louisiana 70775

H. Anne Plettinger  
3456 Villa Rose Drive  
Baton Rouge, Louisiana 70806

President  
West Feliciana Parish Police Jury  
P.O. Box 1921  
St. Francisville, Louisiana 70775

President  
Pointe Coupe Parish Police Jury  
P.O. Box 290  
New Roads, Louisiana 70760

President  
East Feliciana Parish Police Jury  
P.O. Box 427  
Clinton, Louisiana 70722

Parish President  
West Baton Rouge Parish Counsel  
P.O. Box 757  
Port Allen, Louisiana 70767

Mayor-President  
Office of the Mayor-President  
P.O. Box 1471  
Baton Rouge, Louisiana 70821

Asst. Chief Administrative Officer  
P.O. Box 1471  
Baton Rouge, LA 70821

Director  
Dept. of Emergency Management  
East Baton Rouge Parish  
P.O. Box 1471  
Baton Rouge, LA 70821

Director  
Dept. of Emergency Management  
P.O. Box 757  
Port Allen, LA 70767

Secretary Treasurer  
Pointe Coupee Police Jury  
P.O. Box 290  
New Roads, LA 70760

Director  
Dept. of Emergency Management  
P.O. Box 248  
New Roads, LA 70760

Harold B. Ray, Executive Vice President  
Southern California Edison Co.  
San Onofre Nuclear Generating Station  
P.O. Box 128  
San Clemente, California 92674-0128

Chairman, Board of Supervisors  
County of San Diego  
1600 Pacific Highway, Room 335  
San Diego, California 92101

Alan R. Watts, Esq.  
Woodruff, Spradlin & Smart  
701 S. Parker St. Suite 7000  
Orange, California 92868-4720

Sherwin Harris, Resource Project Manager  
Public Utilities Department  
City of Riverside  
3900 Main Street  
Riverside, California 92522

R. W. Krieger, Vice President  
Southern California Edison Company  
San Onofre Nuclear Generating Station  
P.O. Box 128  
San Clemente, California 92674-0128

David Spath, Chief  
Division of Drinking Water and  
Environmental Management  
P.O. Box 942732  
Sacramento, California 94234-7320

Michael R. Olson  
Sr. Energy Administrator  
San Diego Gas & Electric Company  
P.O. Box 1831  
San Diego, California 92112-4150

Mayor  
City of San Clemente  
100 Avenida Presidio  
San Clemente, California 92672

Dwight E. Nunn, Vice President  
Southern California Edison Company  
San Onofre Nuclear Generating Station  
P.O. Box 128  
San Clemente, California 92674-0128

Chief Administrator  
San Diego County  
1600 Pacific Coast Highway  
San Diego, California 92101

Mayor  
City of San Juan Capistrano  
32400 Paseo Adelanto  
San Juan Capistrano, California 92672

City Manager  
City of San Juan Capistrano  
32400 Paseo Adelanto  
San Juan Capistrano, California 92672

City Manager  
City of San Clemente  
100 Avenida Presidio  
San Clemente, California 92672

Mayor  
City of Dana Point  
33282 Golden Lantern  
Dana Point, California 92629

City Manager  
City of Dana Point  
33282 Golden Lantern  
Dana Point, California 92629

William T. Cottle, President and  
Chief Executive Officer  
STP Nuclear Operating Company  
P.O. Box 289  
Wadsworth, Texas 77483

T. H. Cloninger, Vice President  
Engineering & Technical Services  
STP Nuclear Operating Company  
P.O. Box 289  
Wadsworth, Texas 77483

S. M. Head, Supervisor, Licensing  
Quality & Licensing Department  
STP Nuclear Operating Company  
P.O. Box 289  
Wadsworth, Texas 77483

A. Ramirez/C. M. Canady  
City of Austin  
Electric Utility Department  
721 Barton Springs Road  
Austin, Texas 78704

M. T. Hardt/W. C. Gunst  
City Public Service Board  
P.O. Box 1771  
San Antonio, Texas 78296

D. G. Tees/R. L. Balcom  
Houston Lighting & Power Company  
P.O. Box 1700

Houston, Texas 77251

Jon C. Wood  
Matthews & Branscomb  
112 E. Pecan, Suite 1100  
San Antonio, Texas 78205

A. H. Gutterman, Esq.  
Morgan, Lewis & Bockius  
1800 M. Street, N.W.  
Washington, D.C. 20036-5869

G. E. Vaughn/C. A. Johnson  
Central Power & Light Company  
P.O. Box 289  
Mail Code: N5012  
Wadsworth, Texas 77483

INPO  
Records Center  
700 Galleria Parkway  
Atlanta, Georgia 30339-5957

Bureau of Radiation Control  
State of Texas  
1100 West 49th Street  
Austin, Texas 78756

Jim Calloway  
Public Utility Commission  
William B. Travis Building  
P.O. Box 13326  
1701 North Congress Avenue  
Austin, Texas 78701-3326

Judge, Matagorda County  
Matagorda County Courthouse  
1700 Seventh Street  
Bay City, Texas 77414

Mayor, City of Bay City  
City Hall  
1901 Fifth Street  
Bay City, Texas 77414

Mayor, City of Palacios  
P.O. Box 845  
Palacios City Hall  
Palacios, Texas 77465

James Mitchell, Sheriff  
Matagorda County  
2323 Avenue E  
Bay City, Texas 77414

Office of the Governor  
Director, Environmental Policy  
State Liaison Officer  
P.O. Box 12428  
Austin, TX 78756-3189

Charles M. Dugger, Vice President  
Operations - Waterford 3  
Entergy Operations, Inc.  
17265 River Road  
Killona, Louisiana 70066-0751

General Manager, Plant Operations  
Waterford 3 SES  
Entergy Operations, Inc.  
17265 River Road  
Killona, Louisiana 70066-0751

Manager - Licensing Manager  
Waterford 3 SES  
Entergy Operations, Inc.  
17265 River Road  
Killona, Louisiana 70066-0751

Chairman  
Louisiana Public Service Commission  
One American Place, Suite 1630  
Baton Rouge, Louisiana 70825-1697

Director, Nuclear Safety &  
Regulatory Affairs  
Waterford 3 SES  
Entergy Operations, Inc.  
17265 River Road  
Killona, Louisiana 70066-0751

Parish President  
St. Charles Parish  
P.O. Box 302  
Hahnville, Louisiana 70057

President  
St. John the Baptist  
1801 W. Airline Hwy  
LaPlace, Louisiana 70068

Assistant Director  
State Emergency Management  
Military Department  
Office of Emergency Preparedness  
P.O. Box 44217  
Baton Rouge, Louisiana 70804

Chief Operating Officer  
Wolf Creek Nuclear Operating Corp.  
P.O. Box 411  
Burlington, Kansas 66839

Jay Silberg, Esq.  
Shaw, Pittman, Potts & Trowbridge  
2300 N Street, NW  
Washington, DC 20037

Supervisor Licensing  
Wolf Creek Nuclear Operating Corp.  
P.O. Box 411  
Burlington, Kansas 66839

Chief Engineer  
Utilities Division  
Kansas Corporation Commission  
1500 SW Arrowhead Rd.  
Topeka, Kansas 66604-4027

Office of the Governor  
State of Kansas  
Topeka, Kansas 66612

Attorney General  
Judicial Center  
301 S.W. 10th

2nd Floor  
Topeka, Kansas 66612-1597

County Clerk  
Coffey County Courthouse  
110 South 6th Street  
Burlington, Kansas 66839-1798

Frank Moussa  
Division of Emergency Preparedness  
2800 SW Topeka Blvd  
Topeka, Kansas 66611-1287

Coffey County Commissioners  
Coffey County Courthouse  
110 South 6th Street  
Burlington, Kansas 66839

Coffey County Emergency  
Preparedness Coordinator  
Coffey County Courthouse  
110 South 6th Street  
Burlington, Kansas 66839

Mayor, City of Burlington  
City Hall  
P.O. Box 207  
Burlington, Kansas 66839

The Honorable Bill Freeman  
Mayor, City of LeRoy  
406 2nd Street  
LeRoy, Kansas 66857

Mayor, City of New Strawn  
City Hall  
P.O. Box 922  
New Strawn, Kansas 66871

Mayor, City of Waverly  
Rural Route 3, Box 85-B  
Waverly, Kansas 66871

Sheriff  
Allen County Emergency  
Preparedness Coordinator  
Allen County Courthouse

P.O. Box 433  
Iola, Kansas 66749

Lyon County Emergency  
Preparedness Coordinator  
605 Lincoln  
Emporia, Kansas 66801

Anderson County Emergency  
Preparedness Coordinator  
315 Orange Street  
Garnett, Kansas 66032

Public Service Commission  
P.O. Box 360  
Jefferson City, Missouri 65102

Mr. J. V. Parrish (Mail Drop 1023)  
Chief Executive Officer  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968

Chairman  
Energy Facility Site Evaluation Council  
P.O. Box 43172  
Olympia, Washington 98504-3172

Rodney L. Webring (Mail Drop PE08)  
Vice President, Operations Support/PIO  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968

Greg O. Smith (Mail Drop 927M)  
Vice President, Generation  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968

D. W. Coleman (Mail Drop PE20)  
Manager, Regulatory Affairs  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968



Albert E. Mouncer (Mail Drop 1396)  
General Counsel  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968

Paul Inserra (Mail Drop PE20)  
Manager, Licensing  
Energy Northwest  
P.O. Box 968  
Richland, Washington 99352-0968

Bob Nichols  
State Liaison Officer  
Executive Policy Division  
Office of the Governor  
P.O. Box 43113  
Olympia, Washington 98504-3113

Lynn Albin  
Washington State Department of Health  
P.O. Box 7827  
Olympia, WA 98504-7827

John L. Erickson, Director  
Division of Radiation Protection  
Department of Health  
Agricultural Center Building #5  
P.O. Box 47827  
Olympia, Washington 98504-7827

Max E. Benitz, Jr., Chairman  
Board of Benton County Commissioners  
P.O. Box 190  
Prosser, Washington 99350

Sue Miller, Chair  
Board of Franklin County Commissioners  
1016 North 4th Street  
Pasco, Washington 99301

cc via E-mail through ADAMS:

Regional Administrator (**EWM**)

DRP Director (**KEB**)

DRS Director (**ATH**)

DRP Branch Chiefs (**JIT, WDJ, CSM, LJS, WBJ**)

DRP Project Engineers (**DBA, RAK1, RVA, DPL, WCS, LMW1, GAP, JFM1**)

Branch Chief, DRP/TSS (**PHH**)

RITS Coordinator (**NBH**)

Scott Morris (**SAM1**)

NRR Event Tracking System (**IPAS**)

W. Dean (**WMD**)

T. Foley (**TXF1**)

P. Koltay (**PSK**)

D. Hickman (**DEH2**)

D. Coe (**DHC**)

M. Johnson (**MRJ1**)

A. Spector (**AKS**)

R. Blough, RI (**ARB**)

L. Plisco, RII (**LRP**)

G. Grant, RIII (**GEG**)

Site Secretaries (**VLH, DVY, LCA, SLN, AAJ, NJC, MJS, TLB4, PJS, SFN1, LAR, AHY, LEF1, SLA2**)

Senior Resident Inspectors: (**RLB3, VGG, ATG, JAC, DLP, WCW, JLD, JHM2, TWP, JAS7, NFO, TRF, GDR, FLB2**)

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GFLarkin;df	JITapia	WDJohnson	CSMarschall	WBJones
/RA/	/RA/ Performance Indicator Session	/RA/ Insp. Rpt. & Insp. Program Session	/RA/ Assessment & Enforcement Session	/RA/ Risk Insights in Plant Operations Session
1/3/01	1/3/01	1/3/01	1/10/01	1/3/01

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## **ENCLOSURE 1**

### **NRC Sponsored November 15, 2000, Public Workshop Results Summary**

**Description of the Summary:** This summary is organized into four sections. Each section lists the successes and challenges that relate to the initial implementation of the reactor oversight process as discussed in the Public Workshop for each of the panel sessions: Performance Indicators, Significance Determination Process, Inspection Reports and Inspection Program, and Assessment and Enforcement.

#### **1. Performance Indicator's Session Results**

##### **Panel Members:**

Joseph Tapia, NRC, Region IV, DRP Branch Chief  
Charlie Ellsaesser, Corrective Action & Assessment Manager, Grand Gulf  
Donald Hickman, Operations Engineer, NRR Inspection Program Branch  
Steve Floyd, Senior Directory Regulatory Reform & Strategy, Nuclear Energy Institute

##### **A Successes:**

- (1) The FAQ process has been helpful.
- (2) Data has been easy to collect and there is consistency in what is being reported.
- (3) It has focused the industry on risk significance. Examples:
  - a. The security equipment index has reduced the number of compensatory measures and has caused equipment to be fixed more readily
  - b. EP drill participation has resulted in all teams participating in drills
  - c. EP sirens have received more attention
  - d. The downpower PI has had a positive effect on plant equipment performance
- (4) Public confidence has increased as a result of the availability of data to the public. The availability of a comprehensive summary of plant performance for all sites is a positive. Meetings and workshops with public participation to discuss the new program have had a positive effect on public confidence.
- (5) The inspection response to a white PI was viewed as fair and reasonable, confirming the program's predictability and thereby building trust.

**B. Challenges:**

- (1) The number of FAQs is becoming unmanageable. Distinguishing between generic FAQs and plant specific FAQs is difficult.
- (2) Data gathering and record keeping is a challenge. The audit trail for power reductions is an example. The radiation protection PIs require a lot of time to obtain. The verification of the data stream or its audit ability is a challenge. There is a data collection inconsistency in the area of safety system functional failures that stems from 50.73 requirements that results in a majority of BWRs not reporting RCIC failures.
- (3) Unavailability needs a consistent definition. Although guidance exists for taking credit for operator actions, there are apparent inconsistencies in the allowances for operator actions between the PRA and PIs.
- (4) The definition of overhaul hours has led to inconsistencies in the raw data for unavailability.
- (5) The initial use of the WANO methodology has had a negative impact in that work on support systems counts negatively. Examples of this are RHR unavailability and unavailability for other manual and nonautomatic systems. An inconsistency between the Maintenance Rule and PIs in the methodology for accounting of support systems can also be described by saying that inoperability of a support system cascades for PIs and not for PRA nor the Maintenance Rule. Another unintended consequence of this cascading challenge is the “stacking” of work to minimize unavailable time. While the PRA would discourage this practice from a risk insight, the PI would encourage it. Although new Maintenance Rule Section a(4) will serve to counterbalance this dichotomy, it is still a challenge.
- (6) Another mismatch that has presented a challenge is the impact of extended allowed outage times on PIs.
- (7) The white PI thresholds may unnecessarily drive the rescheduling of work. The challenge that results is preventing the improper management of the PId from having negative impact on maintenance.
- (8) A challenge with public perception may arise when all PIs become green. Without a good public understanding of PIs, good performance across the industry may not be recognized as such.

**Performance Indicators Summary:** Overall, PIs are working as expected; however, there does exist a recognized problem in the mitigating systems area. We do have confidence that progress will be made to resolve the remaining issues.

## **2. Significance Determination Process Session Results**

### **Panel Members:**

William Jones, Region IV, DRP Branch Chief

Rick Wachowiak, Supervisor of Risk Management, Cooper

Peter Koltay, Operations Engineer, NRR Inspection Program Branch

Tom Houghton, Senior Project Manager, Nuclear Energy Institute

### **A. Successes:**

- (1) Development of the SDP was an appropriate risk-informed step in the revised oversight process.
- (2) The SDP is effective at evaluating risk.
  - a. The SDP is able to identify safety significance.
  - b. The SDP helps to characterize LERs.
  - c. Many issues are assessed through the SDP without licensee involvement.
  - d. The SDP is shown to identify risk issues.
- (3) NRC and licensees are able to work through SDP.
- (4) Regulatory conference is effective.

### **B. Challenges:**

- (1) Communications:
  - a. When and what are the appropriate discussions between the licensee and the NRC for Phase II and III evaluations?
  - b. How should information identified during the SDP implementation be made available?
  - c. Need to better define the Phase II and Phase III process as it involves licensee/NRC interface.
  - d. We need to clearly define the boundaries of the SDP for findings that should be assessed using this process.
  - e. We need to continue to inform the public that the SDP process is effective. Public meetings are beneficial to inform the public.

- f. We need to establish processes to verify/validate that the SDP is not identifying false negatives and positives
- (2) Better define the scope of the SDP and SERT Panel:
  - a. What resources are needed to implement certain Phase III SDPs? How far do you go to analyze an issue and establish a color?
  - b. We need to continue to use the SDPs to identify areas for improvement.
  - c. How should issues be captured that overlap (i.e., what is the time frame for considering equipment inoperabilities)?
  - d. What is the need for early involvement of the licensee in the SERT panel?
  - e. The NRC needs to be more timely in bringing findings to the SERT and in making its decisions. This includes team inspections findings.
- (3) Appropriate SDPs for Shutdown, Security, and Radiological Protection need to be established.
  - a. There are inconsistencies between the ALARA SDP and the regulations.
  - b. Better definitions need to be used in the SDPs.
  - c. Consider the containment SDP and its actions (minimum yellow).
- (4) Frequently asked questions/responses need to be established for the SDP factoring in how issues are consistently identified and evaluated for the SDP. We need to consider means of evaluating against previous findings.
- (5) Issues which result in no color are not well understood. There are differences between findings that proceed through the SDP and those that do not. The use of no color findings should be eliminated.

### **3. Inspection Reports and Inspection Program Session Results**

#### **Panel Members:**

William Johnson, NRC, Region IV, DRP Branch Chief

Tim Hope, Regulatory Compliance Manager, Comanche Peak

Tom Foley, Operations Engineer, NRR Inspection Program Branch

Steve Floyd, Senior Director Regulatory Reform and Strategy, Nuclear Energy Institute

**A Successes:**

(1) Inspection Reports:

- a. The clarity of the guidance in Manual Chapter 0610\*, "Inspection Reports," has been improved in the areas of thresholds for documentation and handling of minor issues.
- b. Inspection reports are shorter but better focused.
- c. Inspection exit meetings still provide good feedback and inspector insights concerning items which did not meet the documentation threshold.

(2) Inspection Program:

- a. The program is now more predictable.
- b. An overall improvement in communications has been achieved.
- c. Supplemental inspections in response to White performance indicators were properly scoped and well performed.

**B. Challenges:**

(1) Inspection Procedure Content:

- a. The requirements of some inspection procedures exceed the requirements of certain licensees' licensing bases in the fire protection and security areas.
- b. Some of the requirements of the security inspection procedures are unclear.
- c. Review and revise the inspection procedures to remove excessive subjectivity or objectivity (security).

(2) Reduce Unnecessary Regulatory Burden:

- a. Schedule inspections so as to minimize the impact on a particular licensee department by grouping like area inspections.
- b. Allow sufficient lead time for licensees to fulfill information requests prior to team inspections.
- c. Evaluate the volume of information requested of licensees prior to team inspections to ensure that it is really needed.

- d. The lack of flexibility in scheduling of inspections sometimes created difficulties for licensees.

(3) Inspection Resources:

- a. There is a lack of flexibility in applying inspection resources. All plants in the licensee response band get the same level of baseline inspection even if there are differences in performance levels.
- b. There might be too much inspection effort applied to the annual Problem Identification and Resolution team inspection.
- c. The inspection procedure for Access to Radiologically Significant Areas duplicates information provided by the performance indicator, but any findings from this inspection would be less significant than the items counted by the performance indicator.
- d. Consider factoring licensee self-assessments into the program.

(4) Guidance Clarification Needed:

- a. There are conflicts between Manual Chapters 0609 and 0610\*.
- b. Better guidance is needed on the proper treatment of random equipment failures and issues not related to licensee performance deficiencies.
- c. Better guidance is needed on the proper treatment of cross-cutting trends and patterns.
- d. Better guidance is needed on the criteria for no color findings.
- e. Better guidance is needed on the criteria for identifying licensee identified findings and for the proper inspector handling of licensee identified findings.
- f. Continue frequent inspector debriefs with licensee management if the shift to quarterly inspection reports is made.
- g. Management Directive 8.3 is referenced but not issued.



#### **4. Assessment and Enforcement Session Results**

##### **Panel Members:**

Charles Marschall, NRC, Region IV, DRP Branch Chief  
Gregory Gibson, Manager of Projects and Programs, San Onofre  
William Dean, Chief, NRR Inspection Program Branch  
Thomas Houghton, Senior Project Engineer, Nuclear Energy Institute

##### **A Successes:**

- (1) The new process:
  - a. Focuses on risk instead of compliance
  - b. Is less subjective
  - c. Enables licensees to focus on fixing problems in a timely manner verses engaging in “endless dialogue” with the regulator
  - d. Streamlines communications with licensees and stakeholders
  - e. Eliminates responses to Severity Level IV violations that had little or no value added
  - f. Reduced impact on licensees, especially responding to Severity Level IV violations
  - g. Provides a more scrutable assessment process

##### **B. Challenges:**

- (1) The new process does not deal well with cross-cutting issues.
- (2) The Group 1 questions for determining whether issues are minor are not clear.
- (3) The new process can result in double counting of equipment failures.
- (4) Misinterpretation of PI definitions can result in 50.9 issues (the threshold is too low).
- (5) Greater than Green inspection findings and performance indicators should persist for the same period in the assessment process (findings "persist" but PIs do not).
- (6) It is not clear how programmatic issues fit in the process.

- (7) Generally the public in the vicinity of the plants has not shown much interest in attending public meetings.
- (8) Significance thresholds (for color transitions) are important. The nonreactor thresholds don't have the same apparent significance as the reactor thresholds.
- (9) Some participants question the use of delta CDF versus delta CDP for assessing significance.
- (10) The program doesn't clearly address response to plant conditions beyond licensing basis.
- (11) Plant-specific risk models were not developed to a consistent standard.

## Enclosure 2

### Attendees at November 15, 2000, Public Workshop

Name	Title	Organization
Don Allen	Project Engineer	Nuclear Regulatory Commission
Peter Alter	Resident Inspector at Grand Gulf	Nuclear Regulatory Commission
Robert Biggs	Coordinator Licensing Manager	Entergy, River Bend Station
Mike Boyce	-	Cooper Nuclear Station
Ken Brockman	Director, Division of Reactor Projects	Nuclear Regulatory Commission
Frank Brush	Senior Resident Inspector at Wolf Creek	Nuclear Regulatory Commission
Russ Bywater	Senior Resident Inspector at Arkansas Nuclear One	Nuclear Regulatory Commission
Jim Chase	Division Manager of Nuclear Assessment	Fort Calhoun Station
Jeffery Clark	Senior Resident Inspector at Cooper	Nuclear Regulatory Commission
Elmo Collins	Deputy Director, Division of Reactor Projects	Nuclear Regulatory Commission
Linda Cooley	Special Assistant, Division of Resource Management and Resources	Nuclear Regulatory Commission
Mike Cooper	Licensing Specialist	ANO
Kevin Darwich	-	Cooper Nuclear Station
William Dean	Branch Chief, NRR Inspection Program Branch	NRR Inspection Program Office
Pete Donahue	-	Cooper Nuclear Station
Kevin Dorwick	Manager of Performance Analysis	Nebraska Public Power District
Charlie Ellsaesser	Corrective Action & Assessment Manager	Grand Gulf

<b>Name</b>	<b>Title</b>	<b>Organization</b>
Tom Farnholtz	Senior Resident Inspector at Waterford	Nuclear Regulatory Commission
Mary Ferdig	Concerned Citizen	Benedictine University, IL
Steve Floyd	Senior Director Regulatory Reform and Strategy	Nuclear Energy Institute
Tom Foley	Operations Engineer	NRR Inspection Program Office
Mark Frans	Licensing Manager	Ft Calhoun
Denise Freeman	Secretary, Division of Reactor Projects	Nuclear Regulatory Commission
Vincent Gaddy	Senior Resident Inspector at Callaway	Nuclear Regulatory Commission
Greg Gibson	Manager of Projects and Programs	San Onofre
Tony Gody, Jr.	Senior Resident Inspector at Comanche Peak	Nuclear Regulatory Commission
Glenn Griffin	Sr. Staff Nuclear Safety & Licensing Specialist	Entergy Operations
Gilbert Guerra	Resident Inspector at South Texas	Nuclear Regulatory Commission
Charles Hackney	Region IV State Liaison Officer	Nuclear Regulatory Commission
Ken Hall	Supervisor Licensing	Wolf Creek Nuclear Generating Station
John Hanna	Resident Inspector at Callaway	Nuclear Regulatory Commission
Michael Hay	Resident Inspector at Cooper	Nuclear Regulatory Commission
Don Hickman	Operations Engineer	NRR Inspection Program Office
Tim Hope	Regulatory Compliance Manager	Comanche Peak
Tom Houghton	Senior Project Engineer	Nuclear Energy Institute

<b>Name</b>	<b>Title</b>	<b>Organization</b>
Dave Jaffe	Project Manager	NRR
Dale James	Manager of Licensing	ANO
Jodine Jansen	Risk & Reliability Engineer	Comanche Peak
William Johnson	Branch Chief, Reactor Projects	Nuclear Regulatory Commission
Bill Jones	Branch Chief, Reactor Projects	Nuclear Regulatory Commission
Stan Ketelsen	Regulatory Services Supervisor	Diablo Canyon
Robert Kidwell	Emergency Planning	Comanche Peak
Peter Koltay	Operations Engineer	NRR Inspection Program Office
Ron Kopriva	Senior Project Engineer	Nuclear Regulatory Commission
Angela K. Krainik	Director, Regulatory Affairs	Arizona Public Service Company
Grant Larkin	Reactor Engineer	Nuclear Regulatory Commission
Joe Leavines	Manager Licensing	Entergy, River Bend Station
Bill Maier	Senior Engineer	Nuclear Regulatory Commission
Alexander Marion	Director, Programs Nuclear Generation	Nuclear Energy Institute
Carl Markert	Engineering Support Department Manager	Nebraska Public Power District
Charles Marschall	Branch Chief, Reactor Projects	Nuclear Regulatory Commission
C. David Mauldin	Vice President Nuclear Engineering and Support	Arizona Public Service Company
Tony C. McMurtray	RIV Coordinator, Office of the Executive Director for Operations	Nuclear Regulatory Commission

<b>Name</b>	<b>Title</b>	<b>Organization</b>
Ellis W. Merschhoff	Regional Administrator	Nuclear Regulatory Commission
Frank J. Miraglia	Deputy Executive Director for Reactor Programs	Nuclear Regulatory Commission
Robert Moody	Project Manager	NRR
Bill Mookhoek	Licensing	STP Nuclear Operating Company
Dennis Moseley	-	Wolf Creek
Neil O'Keefe	Senior Resident Inspector at South Texas	Nuclear Regulatory Commission
Clyde Osterholtz	Resident Inspector at Fort Calhoun	Nuclear Regulatory Commission
Lucy Owen	Secretary, Division of Reactor Projects	Nuclear Regulatory Commission
John Pellet	Branch Chief	Nuclear Regulatory Commission
Everett P. Perkins, Jr.	Director, Nuclear Safety Assurance	Waterford 3, Entergy Nuclear
Dale Powers	Senior Technical Assistant	Nuclear Regulatory Commission
David Proulx	Senior Resident Inspector at Diablo Canyon	Nuclear Regulatory Commission
Troy Pruett	Senior Resident Inspector at River Bend	Nuclear Regulatory Commission
Gary Randolph	Senior Vice President and CNO	Ameren Union Electric
Bob Reible	Long Range Planning Coordinator (NRC-RROP) Planning Coordinator	Comanche Peak Steam Electric Station
Mark Reidmeyer	Regional Regulatory Affairs Supervisor	Ameren Union Electric
George Replogle	Senior Resident Inspector at WNP2	Nuclear Regulatory Commission
Jerry Roberts	Director, Nuclear Safety Assurance	Grand Gulf

<b>Name</b>	<b>Title</b>	<b>Organization</b>
Dave Robinson	Quality Assessment Manager	Cooper
Nancy Salgado	Resident Inspector at Palo Verde	Nuclear Regulatory Commission
A. Edward Scherer	Manager of Nuclear Oversight and Regulatory Affairs	Southern California Edison
Scott Schwind	Resident Inspector at Comanche Peak	Nuclear Regulatory Commission
Robert Sherman	Principal Engineer	Energy N.W.
Wayne Sifre	Project Engineer	Nuclear Regulatory Commission
Linda Smith	Branch Chief, Reactor Projects	Nuclear Regulatory Commission
Marion E. Smith	Nuclear Quality & Licensing Department Manager	STP Nuclear Operating Company
Douglas W. Snow	Regulatory Compliance Senior Specialist	Comanche Peak
Tom Stetka	Reactor Engineer	Nuclear Regulatory Commission
Ronald D. Stoddard, PE	Consultant	Lincoln Electric System
David Summers	Palo Verde Project Manager	Public Service Company of New Mexico
Joseph Tapia	Branch Chief, Reactor Projects	Nuclear Regulatory Commission
Lance Terry	-	TXU
Randal K. Thorson	Work Control Manager	Nebraska Public Power District
Rick Wachowiak	Supervisor of Risk Management	Cooper
Wayne Walker	Senior Resident Inspector at Fort Calhoun	Nuclear Regulatory Commission
Greg Warnick	Senior Resident Inspector for Palo Verde	Nuclear Regulatory Commission

<b>Name</b>	<b>Title</b>	<b>Organization</b>
Leonard Willoughby	Project Engineer	Nuclear Regulatory Commission
Jennifer Yunk	Program Administrator	Wolf Creek Nuclear Generating Station