

December 20, 1994

Mr. Robert G. Byram  
Senior Vice President-Nuclear  
Pennsylvania Power and Light  
Company  
2 North Ninth Street  
Allentown, PA 18101

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION  
DETERMINATION, AND OPPORTUNITY FOR HEARING, SUSQUEHANNA STEAM  
ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M90731 AND M90732)

Dear Mr. Byram

Enclosed is a copy of the subject notice for your information. This notice  
relates to your application dated October 21, 1994, pertaining to the addition  
of a special test exception to the Technical Specifications for Susuqehanna,  
Units 1 and 2.

This notice has been forwarded to the Office of the Federal Register for  
Publication.

Sincerely,  
/S/

Chester Poslusny, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-387/388

Enclosure: Notice

cc w/encl: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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This notice has been forwarded to the Office of the Federal Register for Publication.

Sincerely,

A handwritten signature in cursive script that reads "Chester Poslusny".

Chester Poslusny, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-387/388

Enclosure: Notice

cc w/encl: See next page

Mr. Robert G. Byram  
Pennsylvania Power & Light Company

Susquehanna Steam Electric Station,  
Units 1 & 2

cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSION  
SUSQUEHANNA STEAM ELECTRIC STATION UNITS 1 & 2  
DOCKET NOS. 50-387 & 50-388  
NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-14 & NPF-22 issued to Pennsylvania Power & Light Company for operation of the Susquehanna Steam Electric Station, Units 1 and 2, located in Luzerne County, Pennsylvania.

The proposed amendment would add the Special Test Exception 3/4.10.6, "Inservice Leak and Hydrostatic Testing," that allows the performance of pressure testing at reactor coolant temperature up to 212°F while remaining in OPERATIONAL CONDITION 4. This special test exception would also require that certain OPERATIONAL CONDITION 3 Specifications for Secondary Containment Isolation, Secondary Containment Integrity and Standby Gas Treatment System operability be met. This change would also revise the Index, Table 1.2, "OPERATIONAL CONDITIONS," and the Bases to incorporate the reference to the proposed special test exception.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes are requested to allow inservice leak and hydrostatic testing, with the reactor in OPERATIONAL CONDITION 4 and the average reactor coolant temperature up to 212°F. The change to allow inservice leak and hydrostatic testing in OPERATIONAL CONDITION 4 will not increase the probability or the consequences of an accident. The probability of a leak in the reactor coolant pressure boundary during inservice leak and hydrostatic testing is not increased by considering the reactor in OPERATIONAL CONDITION 4. The hydrostatic or inservice leak test is performed water solid or near water solid, and temperatures less than or equal to 212°F. The stored energy in the reactor core will be very low and the potential for failed fuel and a subsequent increase in coolant activity above Technical Specification limits are minimal. In addition, secondary containment will be operable and capable of handling airborne radioactivity from leaks that could occur during the performance of hydrostatic or inservice leak testing. Requiring the secondary containment to be operable will ensure that potential airborne radiation from leaks will be filtered through the Standby Gas Treatment System, thus limiting radiation releases to the environment. Therefore, the change will not significantly increase the consequences of an accident.

In the event of a large primary system leak, the reactor vessel would rapidly depressurize allowing the low pressure ECCS systems to operate. The capability of the systems that are required for OPERATIONAL CONDITION 4 would be adequate to keep the core flooded

under this condition. Small system leaks would be detected by leakage inspections before significant inventory loss occurred. This is an integral part of the hydrostatic testing program. Therefore, this change will not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

Allowing the reactor to be considered in OPERATIONAL CONDITION 4 during hydrostatic or leak testing, with a reactor coolant temperature of up to 212°F, is an exception to certain OPERATIONAL CONDITION 3 requirements, including primary containment integrity and total Emergency Core Cooling System operability. The hydrostatic or inservice leakage test is performed water solid, or near water solid, and coolant temperature less than or equal to 212°F. The stored energy in the reactor core will be very low and the potential for failed fuel and a subsequent increase in coolant activity above Technical Specification limits are minimal. In addition, the secondary containment will be operable and capable of handling airborne radioactivity from leaks that could occur during the performance of hydrostatic or inservice leakage testing.

The inservice leak or hydrostatic test conditions remain unchanged. The potential for a system leak remains unchanged since the reactor coolant system is designed for temperatures exceeding 500°F with similar pressures. There are no alternations of any plant systems that cope with the spectrum of accidents. The only difference is that a different subset of systems would be utilized for accident mitigation from those of OPERATIONAL CONDITION 3. Therefore, this change will not create the possibility of a new or different kind of accident from any previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed change allows inservice leak and hydrostatic testing to be performed with a reactor coolant temperature up to 212°F and the reactor in OPERATIONAL CONDITION 4. Since the reactor vessel head will be in place, secondary containment integrity will be maintained and all systems required in OPERATIONAL CONDITION 4 will be operable in accordance with the Technical Specifications, the proposed change will not have any significant impact on any design bases accident or safety limit. The hydrostatic or inservice leak testing is performed water solid, or near water

solid, and temperature less than or equal to 212°F. The stored energy in the core is very low and the potential for failed fuel and a subsequent increase in coolant activity would be minimal. The reactor pressure vessel would rapidly depressurize in the event of a large primary system leak and the low pressure injection systems required to be operable in OPERATIONAL CONDITION 4 would be adequate to keep the core flooded. This would ensure that the fuel would not exceed the 2200°F peak clad temperature limit.

Also requiring secondary containment integrity will assure that potential airborne radiation can be filtered through the SGTS. This will assure that offsite doses remain well within the limits of 10CFR100 guidelines. Small system leaks would be detected by inspections before significant inventory loss could occur. Therefore, this special test exception will not involve a significant reduction in safety margin.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider

all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By January 23, 1995 , the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR

2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, Pennsylvania 18701. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days

prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the

opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to John F. Stolz: petitioner's name and telephone number, date petition was mailed, plant name,

and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Jay Silberg, Esquire, Shaw, Pittman, Potts and Trowbridge, 2300 N Street NW., Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated October 21, 1994, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, Pennsylvania 18701.

Dated at Rockville, Maryland, this 19th day of December 1994.

FOR THE NUCLEAR REGULATORY COMMISSION



Chester Poslusny, Sr. Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation