

December 28, 2000

David J. Modeen, Director
Engineering
Nuclear Generation Division
Nuclear Energy Institute
1776 I Street NW, Suite 400
Washington, DC 20006-3708

Dear Mr. Modeen:

On February 4, 2000, the Nuclear Energy Institute (NEI) provided the NRC staff with its proposed steam generator generic change package, prepared under the industry initiative NEI 97-06. The NEI 97-06 document, *Steam Generator Program Guidelines*, references several Electric Power Research Institute guideline documents that provide the technical details of the proposed industry program for managing SG integrity. At the time of the Indian Point Unit 2 steam generator failure last February, NRC and NEI had made significant progress toward developing a revised regulatory framework for assuring steam generator integrity based on NEI 97-06. The staff suspended its review of the NEI 97-06 as a result of regulatory activities associated with the Indian Point Unit 2 steam generator tube rupture on February 15, 2000.

We plan to resume work on NEI 97-06 in the near future. After we complete our review of the generic change package, we plan to prepare a safety evaluation (SE) related to the revised SG regulatory framework. This SE will provide background on the existing regulatory framework and the reasons why it needs to be revised. The SE will contain a technical evaluation of the proposed new regulatory framework. Of necessity, the staff will need to receive the current revisions of the Electric Power Research Institute guideline documents since we anticipate the SE on NEI 97-06 will discuss aspects of the EPRI reports referenced in NEI 97-06. As we have previously indicated in public meetings and discussions with NEI representatives, the SE will be issued for public comments and will be reviewed by the Advisory Committee on Reactor Safeguards and by the Committee for Review of Generic Requirements. At the conclusion of this process, the staff will also provide a briefing for the NRC Commissioners on this subject.

Organizations that request their documents be handled in a proprietary fashion must follow the requirements of 10 CFR 2.790 which, in general, requires a non-proprietary version of the report be available to the public.

Our understanding is that the EPRI reports referenced in NEI 97-06 contain information that EPRI considers to be proprietary in nature. For the staff to successfully complete the actions described above related to the NEI 97-06 initiative, we will need to have both proprietary and non-proprietary versions of the referenced EPRI reports.

David J. Modeen

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We would appreciate a response from you on this request and a schedule that you can support for providing these reports. If you have any questions on this request, please contact Robert Rothman at (301) 415-3306.

Your cooperation on this important initiative is greatly appreciated.

Sincerely,

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Brian W. Sheron, Associate Director
for Project Licensing and Technical Analysis
Office of Nuclear Reactor Regulation

David J. Modeen

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Brian W. Sheron, Associate Director
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