January 8, 2001

Mr. Harold W. Keiser
Chief Nuclear Officer & President
PSEG Nuclear LLC - X04
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION, CHANGES TO TECHNICAL SPECIFICATION BASES (TAC NO. MB0798)

Dear Mr. Keiser:

In a letter dated December 11, 2000, PSEG Nuclear LLC provided to the U.S. Nuclear Regulatory Commission (NRC) revised Technical Specification (TS) Bases page B 3/4 1-3 for the Hope Creek Generating Station (HCGS). We understand that this change is being made pursuant to 10 CFR 50.59 and does not require NRC review and approval. Therefore, we have not reviewed the change and issuance of the revised TS Bases does not constitute NRC approval. The enclosed TS page is being distributed for inclusion in the HCGS TSs.

Sincerely,

/RA/

Richard B. Ennis, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure: TS Page B 3/4 1-3

cc w/encl: See next page

Hope Creek Generating Station

cc:

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U.S. Nuclear Regulatory Commission
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c/o Mary O. Henderson, Clerk
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3/4.1.4 CONTROL ROD PROGRAM CONTROLS

Control rod withdrawal and insertion sequences are established to assure that the maximum insequence individual control rod or control rod segments which are withdrawn at any time during the fuel cycle could not be worth enough to result in peak fuel enthalpy greater than 280 cal/gm in the event of a control rod drop accident. The specified sequences are characterized by homogeneous, scattered patterns of control rod withdrawal. When THERMAL POWER is greater than 10% of RATED THERMAL POWER, there is no possible rod worth which, if dropped at the design rate of the velocity limiter, could result in a peak enthalpy of 280 cal/gm. Thus requiring the RWM to be OPERABLE when THERMAL POWER is less than or equal to 10% of RATED THERMAL POWER provides adequate control.

The RWM provides automatic supervision to assure that out-of-sequence rods will not be withdrawn or inserted.

The analysis of the rod drop accident is presented in Section 15.4.9 of the FSAR and the techniques of the analysis are presented in a topical report, Reference 1.

The RBM is designed to automatically prevent fuel damage in the event of erroneous rod withdrawal from locations of high power density during high power operation. Two channels are provided. Tripping one of the channels will block erroneous rod withdrawal soon enough to prevent fuel damage. This system backs up the written sequence used by the operator for withdrawal of control rods. Operability of a RBM channel is assured for a given control rod when $\geq$ 50% of the LPRM inputs for each detector level are available for that rod. When < 50% of the LPRM inputs on either detector level are available, a case-by-case evaluation of channel operability is required.