

December 23, 2000

SVP-00-191

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Quad Cities Generating Station, Unit 1
Facility Operating License No. DPR-29
NRC Docket No. 50-254

Subject: Licensee Event Report Concerning Automatic Reactor Scram
from Low Reactor Vessel Level

Enclosed is Licensee Event Report (LER) 254/00-010, "Licensee Event Report Concerning Automatic Reactor Scram from Low Reactor Vessel Level" Revision 00, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(iv), which requires reporting of any operation or condition prohibited by the Plant's Technical Specifications

We are committing to the following actions:

The root cause determination for this event is in progress but has not been finalized. Upon completion of the root cause investigation, a supplemental report will be issued

Any other actions described in the submittal represent intended or planned actions by Commonwealth Edison (ComEd) Company. They are described for the NRC's information and are not regulatory commitments.

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Should you have any questions concerning this letter, please contact Mr. C.C. Peterson at (309) 654-2241, extension 3609.

Respectfully,

A handwritten signature in cursive script, reading "Joel P. Dimmette, Jr.".

Joel P. Dimmette, Jr.
Site Vice President
Quad Cities Generating Station

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Quad Cities Generating Station

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bcc: Project Manager – NRR
Office of Nuclear Facility Safety, - IDNS
Senior Reactor Analyst – IDNS
Resident Inspector - IDNS
Manager of Energy Practice – Winston and Strawn
Director, Licensing and Compliance – ComEd
Vice President, Regulatory Services– ComEd
ComEd Document Control Desk Licensing (Hard Copy)
ComEd Document Control Desk Licensing (Electronic Copy)
W. Leech – MidAmerican Energy Company
D. Tubbs – MidAmerican Energy Company
Regulatory Assurance Manager – Dresden Generating Station
Regulatory Assurance Manager – Quad Cities Generating Station
NRC Coordinator – Quad Cities Generating Station
NSRB Site Coordinator – Quad Cities Generating Station
Site Vice President – Quad Cities Generating Station
Station Manager – Quad Cities Generating Station
SVP Letter File

| NRC FORM 366 (6-1998) | | | | U.S. NUCLEAR REGULATORY COMMISSION | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
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| LICENSEE EVENT REPORT (LER) | | | | APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001 <small>Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the information and Records Management Branch (t-6 133), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office Of Management And Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.</small> | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| FACILITY NAME (1) Quad Cities Nuclear Power Station, Unit 1 | | | | DOCKET NUMBER (2) 05000254 | | PAGE (3) 1 of 3 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| TITLE (4) Automatic Reactor Scram from Low Reactor Vessel Level | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| EVENT DATE (5) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th style="width:10%;">MON TH</th> <th style="width:10%;">DAY</th> <th style="width:10%;">YEAR</th> </tr> <tr> <td style="text-align: center;">12</td> <td style="text-align: center;">06</td> <td style="text-align: center;">2000</td> </tr> </table> | | | MON TH | DAY | YEAR | 12 | 06 | 2000 | LER NUMBER (6) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th style="width:10%;">YEAR</th> <th style="width:10%;">SEQUENTIAL NUMBER</th> <th style="width:10%;">REVISION NUMBER</th> </tr> <tr> <td style="text-align: center;">2000</td> <td style="text-align: center;">010</td> <td style="text-align: center;">00</td> </tr> </table> | | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | 2000 | 010 | 00 | REPORT DATE (7) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th style="width:10%;">MONTH</th> <th style="width:10%;">DAY</th> <th style="width:10%;">YEAR</th> </tr> <tr> <td style="text-align: center;">12</td> <td style="text-align: center;">22</td> <td style="text-align: center;">2000</td> </tr> </table> | | | MONTH | DAY | YEAR | 12 | 22 | 2000 | OTHER FACILITIES INVOLVED (8) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th style="width:50%;">FACILITY NAME</th> <th style="width:50%;">DOCKET NUMBER</th> </tr> <tr> <td style="text-align: center;">N/A</td> <td style="text-align: center;">05000</td> </tr> <tr> <th style="width:50%;">FACILITY NAME</th> <th style="width:50%;">DOCKET NUMBER</th> </tr> <tr> <td style="text-align: center;">N/A</td> <td style="text-align: center;">05000</td> </tr> </table> | | FACILITY NAME | DOCKET NUMBER | N/A | 05000 | FACILITY NAME | DOCKET NUMBER | N/A | 05000 |
| MON TH | DAY | YEAR | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 12 | 06 | 2000 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 2000 | 010 | 00 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| MONTH | DAY | YEAR | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 12 | 22 | 2000 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| FACILITY NAME | DOCKET NUMBER | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| N/A | 05000 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| FACILITY NAME | DOCKET NUMBER | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| N/A | 05000 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| OPERATING MODE (9) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:10%; text-align: center;">1</td> </tr> </table> | | | 1 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more) (11) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 1 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| POWER LEVEL (10) <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:10%; text-align: center;">100</td> </tr> </table> | | | 100 | 20.2201(b) | | 20.2203(a)(2)(v) | | 50.73(a)(2)(i) | | 50.73(a)(2)(viii) | | | | | | | | | | | | | | | | | | | | | | | | | | |
| | | | 100 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| | | | 20.2203(a)(i) | | 20.2203(a)(3)(i) | | 50.73(a)(2)(ii) | | 50.73(a)(2)(x) | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| | | | 20.2203(a)(2)(i) | | 20.2203(a)(3)(ii) | | 50.73(a)(2)(iii) | | 73.71 | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| | | | 20.2203(a)(2)(ii) | | 20.2203(a)(4) | | 50.73(a)(2)(iv) | | OTHER | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 20.2203(a)(2)(iii) | | 50.36(c)(1) | | 50.73(a)(2)(v) | | Specify in Abstract below or in NRC Form 365A | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 20.2203(a)(2)(iv) | | 50.36(c)(2) | | 50.73(a)(2)(vii) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| NAME Charles Peterson, Regulatory Assurance Manager | | | | | | TELEPHONE NUMBER (Include Area Code) (309) 654-2241 ext 3609 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| CAUSE | | SYSTEM | | COMPONENT | | MANUFACTURER | | REPORTABLE TO EPIX | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
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| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| X YES <small>(If yes, complete EXPECTED SUBMISSION DATE)</small> | | | | NO | | | | EXPECTED SUBMISSION DATE (15) | | <table border="1" style="width:100%; border-collapse: collapse;"> <tr> <th style="width:10%;">MONTH</th> <th style="width:10%;">DAY</th> <th style="width:10%;">YEAR</th> </tr> <tr> <td style="text-align: center;">02</td> <td style="text-align: center;">07</td> <td style="text-align: center;">2001</td> </tr> </table> | | MONTH | DAY | YEAR | 02 | 07 | 2001 | | | | | | | | | | | | | | | | | | | |
| MONTH | DAY | YEAR | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| 02 | 07 | 2001 | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| ABSTRACT (Limit to 1400 spaces, i. e., approximately 15 single-spaced typewritten lines) (16) | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
| <p>At 0940 hours on December 6, 2000, with reactor power at 100%, Unit One reactor water level started to decrease and the low level alarm was received. The operators took the Feedwater Level Control System (FWLC) out of three-element control and placed it in single-element control. They then took manual control of the 'B' Feedwater Regulating Valve (FWRV) and opened the valve, causing reactor level to increase. At about 0941 hours reactor level started to decrease rapidly and the 'A' FWRV locked up. A scram signal was received at 0942 hours, with indicated reactor water level at 20 inches. All automatic actions occurred consistent with a low-level trip.</p> <p>The root cause determination for this event is in progress but has not been finalized. Upon completion of the root cause investigation, a supplemental report will be issued.</p> <p>The safety significance of this event was minimal. Although the event resulted in a reactor trip, all safety systems responded as designed.</p> | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |

LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET (2) | LER NUMBER (6) | | | PAGE (3) |
|-------------------------------------------|------------|----------------|-------------------|-----------------|----------|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| Quad Cities Nuclear Power Station, Unit 1 | 05000254 | 2000 | 010 | 00 | 2 of 3 |

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power
Energy Industry Identification System (EIIIS) Codes are identified in the text as [XX] and are obtained from IEEE Standard 805-1984, IEEE Recommended Practice for System Identification in Nuclear Power Plants and Related Facilities.

EVENT IDENTIFICATION:

Automatic Reactor Scram from Low Reactor Vessel Level

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 1 Event Date: December 6, 2000 Event Time: 0942 hours
Reactor Mode: 1 Mode Name: Power Operation Power Level: 100%

Power Operation (1) - Mode switch in the RUN position with average reactor coolant temperature at any temperature.

B. DESCRIPTION OF EVENT:

At 0940 hours on December 6, 2000, with reactor power at 100%, Unit One reactor water level started to decrease and the low level alarm was received. The operators took the Feedwater Level Control System (FWLC) [JB] out of three-element control and placed it in single-element control. They then took manual control of the 'B' Feedwater Regulating Valve (FWRV) [LCV] and opened the valve, causing reactor level to increase. At about 0941 hours reactor level started to decrease rapidly and the 'A' FWRV locked up. A scram signal [JC] was received at 0942 hours, with indicated reactor water level at 20 inches. All automatic actions occurred consistent with a low-level trip.

C. CAUSE OF EVENT:

The root cause determination for this event is in progress but has not been finalized. Upon completion of the root cause investigation, a supplemental report will be issued.

D. SAFETY ANALYSIS

The safety significance of this event was minimal. Although the event resulted in a reactor trip, all safety systems responded as designed.

E. CORRECTIVE ACTIONS:

The root cause determination for this event is in progress but has not been finalized. Upon completion of the root cause investigation, a supplemental report will be issued.

LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | | DOCKET (2) | LER NUMBER (6) | | | PAGE (3) |
|-------------------------------------------|--|------------|----------------|----------------------|--------------------|----------|
| Quad Cities Nuclear Power Station, Unit 1 | | 05000254 | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | 3 of 3 |
| | | | 2000 | 010 | 00 | |

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

F. PREVIOUS OCCURRENCES:

Previous occurrences will be provided in the supplemental report.

G. COMPONENT FAILURE DATA:

Component failure data will be provided in the supplemental report.