February 28, 19

Mr. Robert G. Byram Senior Vice President-Nuclear Pennsylvania Power and Light Company 2 North Ninth Street Allentown, PA 18101

SUBJECT: REVISIONS TO SURVEILLANCE REQUIREMENT 4.0.5, SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M89826 and M89827)

Dear Mr. Byram:

The Commission has issued the enclosed Amendment No.  $^{144}$  to Facility Operating License No. NPF-14 and Amendment No.  $^{113}$  to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2. These amendments are in response to your letter dated June 23, 1994.

These amendments revise Technical Specification 4.0.5, which provides the requirements for inservice inspection and testing of ASME Code components, to conform to Standard Technical Specifications (NUREG-1433).

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly <u>Federal</u> <u>Register</u> notice.

Sincerely,
/S/

Chester Poslusny, Senior Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-387/50-388

Enclosures:

- 1. Amendment No. 144 to License No. NPF-14
- 2. Amendment No. 113 to License No. NPF-22
- 3. Safety Evaluation

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WASHINGTON, D.C. 20555-0001

February 28, 1995

Mr. Robert G. Byram Senior Vice President-Nuclear Pennsylvania Power and Light Company 2 North Ninth Street Allentown, PA 18101

SUBJECT: REVISIONS TO SURVEILLANCE REQUIREMENT 4.0.5, SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M89826 and M89827)

Dear Mr. Byram:

The Commission has issued the enclosed Amendment No.  $^{144}$  to Facility Operating License No. NPF-14 and Amendment No.  $^{113}$  to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2. These amendments are in response to your letter dated June 23, 1994.

These amendments revise Technical Specification 4.0.5, which provides the requirements for inservice inspection and testing of ASME Code components, to conform to Standard Technical Specifications (NUREG-1433).

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Chester Poslusny, Senior Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-387/50-388

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- 1. Amendment No. 144 to License No. NPF-14
- 2. Amendment No. 113 to License No. NPF-22
- 3. Safety Evaluation

cc w/encls: See next page Mr. Robert G. Byram Pennsylvania Power & Light Company

#### cc:

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Pennsylvania Department of
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# UNITED STATES NUCLEAR REGULATORY COMMISSION PHILADELPHIA ELECTRIC COMPANY LIMERICK GENERATING STATION, UNITS 1 AND 2 DOCKET NOS. 50-352 AND 50-353 ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-85, issued to Philadelphia Electric Company, (the licensee), for operation of the Limerick Generating Station, Units 1 and 2, located in Montgomery County, Pennsylvania. <u>ENVIRONMENTAL ASSESSMENT</u>

### Identification of the Proposed Action:

This Environmental Assessment has been prepared to address potential environmental issues related to the licensee's application of December 9, 1993, as supplemented July 5, September 9, October 19, November 19, 1994, and January 6, and January 23, 1995, to amend the Limerick Generating Station (LGS), Units 1 and 2 operating licenses. The proposed amendment would increase the licensed thermal power level from 3293 Mwt to 3458 Mwt. This request is in accordance with the generic boiling water reactor (BWR) power uprate program established by the General Electric Company (GE) and approved by U.S. Nuclear Regulatory Commission (NRC) staff in a letter of September 30, 1991.

The proposed action involves NRC issuance of a license amendment to increase the authorized power level by changing the operating license, including Appendix A of the license (Technical Specifications). No change is

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WASHINGTON, D.C. 20555-0001

## PENNSYLVANIA POWER & LIGHT COMPANY

## ALLEGHENY ELECTRIC COOPERATIVE, INC.

## DOCKET NO. 50-387

## SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. <sup>144</sup> License No. NPF-14

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated June 23, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:
  - (2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 144 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and is to be implemented within 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION J.M. Duffin For

John F. Stolz, Director Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 28, 1995

## ATTACHMENT TO LICENSE AMENDMENT NO. 144

## FACILITY OPERATING LICENSE NO. NPF-14

### DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE	<u>INSERT</u>
3/4 0-2	3/4 0-2
3/4 0-3	3/4 0-3
B 3/4 0-6	B 3/4 0-6

#### SURVEILLANCE REQUIREMENTS

- 4.0.1 Surveillance Requirements shall be met during the OPERATIONAL CONDITIONS or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.
- 4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.
- 4.0.3 Failure to perform a Surveillance Requirement within the <u>allowed surveillance</u> interval, <u>defined by Specification 4.0.2</u>, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. <u>The time</u> limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than <u>24 hours</u>. Surveillance Requirements do not have to be performed on inoperable equipment.
- 4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements.
- 4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, & 3 components shall be applicable as follows:
  - a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a.
  - b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda are as follows.

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#### SURVEILLANCE REQUIREMENTS (Continued)

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities	Required frequencies for performing inservice inspection and testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days
Biennially or every 2 years	At least once per 731 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. The provisions of Specification 4.0.3 are applicable to inservice testing activities.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC Staff position on Schedule, Methods and Personnel, and sample expansions included in the Generic Letter.

#### 3/4.0 APPLICABILITY

#### BASES (Cont'd)

Under the provisions of this specification, the applicable Surveillance Requirements must be performed within the specified surveillance interval to assume that the Limiting Conditions for Operation are met during initial plant startup or following a plant outage.

When a shutdown is required to comply with ACTION requirements, the provisions of Specification 4.0.4 do not apply because this would delay placing the facility in a lower CONDITION of operation.

<u>Specification 4.0.5</u> establishes the requirement that inservice inspection of ASME Code Class 1,2, and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves shall be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50.55a.

This specification includes a clarification of the frequencies for performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout the Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda. The requirements of Specification 4.0.4 to perform surveillance activities before entry into an OPERATIONAL CONDITION or other specified condition takes precedence over the ASME Boiler and Pressure Vessel Code provision that allows pumps and valves to be tested up to one week after return to normal operation. The Technical Specification definition of OPERABLE does not allow a grace period before a component, which is not capable of performing its specified function, is declared inoperable and takes precedence over the ASME Boiler and Pressure Vessel Code provision that allows a valve to be incapable of performing its specified function for up to 24 hours before being declared inoperable.



WASHINGTON, D.C. 20555-0001

# PENNSYLVANIA POWER & LIGHT COMPANY

# ALLEGHENY ELECTRIC COOPERATIVE, INC.

## DOCKET NO. 50-388

## SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. <sup>113</sup> License No. NPF-22

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated June 23, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:
  - (2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 113 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and is to be implemented within 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION J. le ( Jelishan for

John F. Stolz, Director Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 28, 1995

# ATTACHMENT TO LICENSE AMENDMENT NO. 113

## FACILITY OPERATING LICENSE NO. NPF-22

## DOCKET NO. 50-388

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
3/4 0-2	3/4 0-2
3/4 0-3	3/4 0-3
B 3/4 0-6	B 3/4 0-6

#### SURVEILLANCE REQUIREMENTS

- 4.0.1 Surveillance Requirements shall be met during the OPERATIONAL CONDITIONS or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.
- 4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.
- 4.0.3 Failure to perform a Surveillance Requirement within the <u>allowed surveillance</u> interval, <u>defined by Specification 4.0.2</u>, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. <u>The time</u> limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.
- 4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements.
- 4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, & 3 components shall be applicable as follows:
  - a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a.
  - b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda are as follows.

## SURVEILLANCE REQUIREMENTS (Continued)

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities	Required frequencies for performing inservice inspection and testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days
Biennially or every 2 years	At least once per 731 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. The provisions of Specification 4.0.3 are applicable to inservice testing activities.
- f. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC Staff position on Schedule, Methods and Personnel, and sample expansions included in the Generic Letter.

#### 3/4.0 APPLICABILITY

#### BASES (Cont'd)

Specification 4.0.4 establishes the requirement that all applicable surveillances must be met before entry into an OPERATIONAL CONDITION or other condition of operation specified in the Applicability statement. The purpose of this specification is to ensure that system and component OPERABILITY requirements or parameter limits are met before entry into an OPERATIONAL CONDITION or other specified condition for which these systems and components ensure safe operation of the facility. This provision applies to changes in OPERATIONAL CONDITIONS or other specified conditions associated with plant shutdown as well as startup.

Under the provisions of this specification, the applicable Surveillance Requirements must be performed within the specified surveillance interval to assume that the Limiting Conditions for Operation are met during initial plant startup or following a plant outage.

When a shutdown is required to comply with ACTION requirements, the provisions of Specification 4.0.4 do not apply because this would delay placing the facility in a lower CONDITION of operation.

<u>Specification 4.0.5</u> establishes the requirement that inservice inspection of ASME Code Class 1,2, and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves shall be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50.55a.

This specification includes a clarification of the frequencies for performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout the Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda. The requirements of Specification 4.0.4 to perform surveillance activities before entry into an OPERATIONAL CONDITION or other specified condition takes precedence over the ASME Boiler and Pressure Vessel Code provision that allows pumps and valves to be tested up to one week after return to normal operation. The Technical Specification definition of OPERABLE does not allow a grace period before a component, which is not capable of performing its specified function, is declared inoperable and takes precedence over the ASME Boiler and Pressure Vessel Code provision that allows a valve to be incapable of performing its specified function for up to 24 hours before being declared inoperable.



WASHINGTON, D.C. 20555-0001

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO.<sup>144</sup>TO FACILITY OPERATING LICENSE NO. NPF-14 AMENDMENT NO.<sup>113</sup> TO FACILITY OPERATING LICENSE NO. NPF-22 PENNSYLVANIA POWER & LIGHT COMPANY

# ALLEGHENY ELECTRIC COOPERATIVE, INC.

# SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

DOCKET NOS. 50-387 AND 388

## 1.0 INTRODUCTION

By letter dated June 23, 1994, the Pennsylvania Power and Light Company (the licensee) submitted a request for changes to the Susquehanna Steam Electric Station (SSES), Units 1 and 2, Technical Specifications (TS). The requested changes would revise Technical Specification 4.0.5, which provides the requirements for inservice inspection and testing of ASME Code components, to conform to Standard Technical Specifications (STS, i.e. NUREG-1433).

### 2.0 EVALUATION

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TS 4.0.5 contains surveillance requirements for inservice inspection (ISI) and testing of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) components. Presently, TS 4.0.5.a states that specific written relief from the Commission is required for relief requests. This specification could be interpreted to be more restrictive than the regulations in 10 CFR 50.55a. In order to avoid any misinterpretations, the licensee proposes to revise TS 4.0.5 for both units to conform to the Standard Technical Specifications (NUREG-1433). The following is a description of the proposed changes:

- (1) Revise item a of TS 4.0.5 by deleting "(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55 a(g)(6)(i)."
- (2) Revise item b of TS 4.0.5 to read ".....Pressure Vessel Code and applicable Addenda are as follows" from the present wording of ".....Pressure Vessel Code and applicable addenda shall be applicable as follows in these Technical Specifications".
- (3) Add "Biennially or every 2 years" and "At least once per 731 days" to item b in the frequency table.

- (4) Replace item d of TS 4.0.5 "Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements," with the statement "The provisions of Specification 4.0.3 are applicable to inservice testing activities."
- (5) Revise Bases Section, page B 3/4 0-6, for Specification 4.0.5, by deleting the statement, "these requirements apply except when relief has been provided in writing by the Commission."
- (6) Unit 2 only, delete the \*\* footnote on page 3/4 0-2 which is no longer applicable to Unit 2.

The proposed change (1) conforms to STS section 5.7.2.12.a, proposed changes (2) and (3) conform to STS section 5.7.2.12.b, and proposed change (4) conforms to STS section 5.7.2.12.d. The proposed change (5) removes the requirement to have NRC approval of reliefs from the ASME code of record and is consistent with change (1) and the proposed change (6) is an administrative change only.

The Standard Technical Specifications (NUREG-1433) reflect the staff's position that the licensee must establish and implement the program in accordance with 10 CFR 50.55a, which permits a licensee to inform the staff of the ASME Code requirements which are impractical and to request relief up to 12 months after the beginning of the updated interval. If later in the interval, the licensee finds a specific need for relief, the request should be submitted for NRC approval after identification of the impractical requirement.

For 120-month updated programs, relief requests should be submitted prior to the interval start date to allow a period for NRC review 12 months after the interval start date (i.e., submit the updated program 3 to 6 months prior to the start date, or earlier). Upon determining an impractical requirement and not including that requirement in the revised inservice test or inspection program, the licensee must follow the requirements of 10 CFR 50.55a(f)(5)(iv) or (g)(5)(iv), as applicable. The change to the specification does not allow the licensee to implement alternative testing under 10 CFR 50.55a, paragraphs (a)(3)(i) and (a)(3)(ii), until the NRC has determined that such alternatives are authorized, and has issued a safety evaluation to the licensee. However, this TS change will enable licensees to avoid situations where compliance with the current TS cannot be achieved, for the period between preparation and submittal of a relief request as part of a revised inservice test or inspection program during the first 12 months of the program, and issuance of the NRC's safety evaluation and granting of relief. This situation could occur at the beginning of a new interval.

Following implementation of the TS change, when a Code requirement is practical, but an alternate method is requested, approval from the NRC is required before implementing the alternative method of testing (1) proposed to achieve levels of quality and safety equivalent to those of the Code method or (2) proposed to avoid an undue hardship without yielding a compensating increase in the level of quality and safety. Additionally, for inservice testing, the licensee may use the guidance in Generic Letter (GL) 89-04, "Guidance on Developing Acceptable Inservice Testing Programs," for alternatives that the staff has determined are acceptable for implementation.

The staff agrees that the proposed changes are administrative in nature. The proposed revision to TS 4.0.5 eliminates the possibility of misinterpretation of the ASME Code requirements. It is permissible for a utility to submit relief requests that are based on impracticality to the NRC within 1 year of the beginning of the updated interval and to implement these requests prior to NRC review and approval. Further, the NRC will continue to provide acceptance of the relief requests in writing.

Therefore, the staff concludes that the proposed revision to TS 4.0.5 for SSES, Units 1 and 2 to reflect the Standard TS section 5.7.2.12 in order to avoid any misinterpretations is acceptable. The staff finds the change to the Bases section and the administrative deletion of the footnote on page 3/4 O-2 also to be acceptable.

#### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 39595). Accordingly, the amendments meet eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Liu P. Campbell

Date: February 28, 1995