Docket Nos. 50-387/388

Mr. Harold W. Keiser Senior Vice President-Nuclear Pennsylvania Power and Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

Dear Mr. Keiser:

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SUBJECT: LICENSED OPERATOR TRAINING AND REQUALIFICATION, SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. 76813 and 76814)

The Commission has issued the enclosed Amendment No. 111 to Facility Operating License No. NPF-14 and Amendment No. 80 to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2. These amendments are in response to your letter dated April 2, 1990.

These amendments would change Technical Specification 6.4 for both Units 1 and 2. The phrase "and Appendix 'A' of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience" is being deleted. Instead, the phrase "except that the licensed operator initial training and requalification programs shall meet or exceed the requirements of 10 CFR Part 55 and utilize the guidance contained in Regulatory Guide 1.8, Rev. 2" is being added.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

/S/

James J. Raleigh, Acting Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

#### Enclosures:

 Amendment No. 111 to License No. NPF-14

2. Amendment No. 80 to License No. NPF-22

Safety Evaluation

cc w/enclosures: See next page

OFC : PDT-2/AA : PDT-2/PM : LHFB : PDT-2/D : OGC

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WASHINGTON, D.C. 20665

June 26, 1991

Reissland with updated overleaf pages 6-8 for both units, 1 and 2, to reflect amendments 108 and 77 respectively.

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Sincerely,

James J. Raleigh, Acting Project Manager
Project Directorate 1-2

Project Directorate I-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 111 to License No. NPF-14

2. Amendment No. 80 to License No. NPF-22

3. Safety Evaluation

cc w/enclosures: See next page Mr. Harold W. Keiser Pennsylvania Power & Light Company

cc:

Jay Silberg, Esq. Shaw, Pittman, Potts & Trowbridge 2300 N Street N.W. Washington, D.C. 20037

Bryan A. Snapp, Esq. Assistant Corporate Counsel Pennsylvania Power & Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

Mr. J. M. Kenny Licensing Group Supervisor Pennsylvania Power & Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

Mr. Scott Barber Senior Resident Inspector U. S. Nuclear Regulatory Commission P.O. Box 35 Berwick, Pennsylvania 18603-0035

Mr. Thomas M. Gerusky, Director Bureau of Radiation Protection Resources Commonwealth of Pennsylvania P. 0. Box 2063 Harrisburg, Pennsylvania 17120

Mr. Jesse C. Tilton, III Allegheny Elec. Cooperative, Inc. 212 Locust Street P.O. Box 1266 Harrisburg, Pennsylvania 17108-1266 Susquehanna Steam Electric Station Units 1 & 2

Mr. S. B. Ungerer
Joint Generation
Projects Department
Atlantic Electric
P.O. Box 1500
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Regional Administrator, Region I U.S. Muclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406

Mr. Harold G. Stanley
Superintendent of Plant
Susquehanna Steam Electric Station
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. Herbert D. Woodeshick Special Office of the President Pennsylvania Power and Light Company 1009 Fowles Avenue Berwick, Pennsylvania 18603

Mr. Robert G. Byram Vice President-Nuclear Operations Pennsylvania Power and Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

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WASHINGTON, D.C. 20555

### PENNSYLVANIA POWER & LIGHT COMPANY

### ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

# SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111 License No. NPF-14

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated April 2, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:
  - (2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 111 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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This license amendment is effective 30 days after its date of issuance. 3.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director Project Directorate I-2 Division of Reactor Projects - I/II

Attachment: Changes to the Technical Specifications

Date of Issuance: June 26, 1991

# ATTACHMENT TO LICENSE AMENDMENT NO.111

# FACILITY OPERATING LICENSE NO. NPF-14

## DOCKET MO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised page is identified by Amendment number and contain vertical lines indicating the area of change. The overleaf page is provided to maintain document completeness.\*

REMOVE	INSERT
6-7	6-7
6-8	6-8*

1

# 6.2.3 NUCLEAR SAFETY ASSESSMENT GROUP (NSAG)

### **FUNCTION**

6.2.3.1 The NSAG shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety.

### COMPOSITION

6.2.3.2 The NSAG shall be composed of at least five dedicated, full-time engineers with at least three located onsite, each with a bachelor's degree in engineering or related science and at least two years professional level experience in his field, at least one year of which experience shall be in the nuclear field.

### RESPONSIBILITIES

6.2.3.3 The NSAG shall be responsible for maintaining surveillance of unit activities to provide independent verification\* that these activities are performed correctly and that human errors are reduced as much as practical.

### **AUTHORITY**

6.2.3.4 The NSAG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving unit safety to the Senior Vice President-Nuclear.

## 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit.

## 6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for the Radiological Protection Supervisor or Health Physics/Chemistry Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the shift Technical Advisor who shall meet or exceed the qualifications referred to in Section 2.2.1.b of Enclosure 1 of the October 30, 1979 NRC letter to all operating nuclear power plants.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager - Nuclear Training, and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 except that the licensed operator initial training and requalification programs shall meet or exceed the requirements of 10 CFR 55 and utilize the guidance contained in Regulatory Guide 1.8 Rev. 2.

<sup>\*</sup>Not responsible for sign-off function.

### **6.5 REVIEW AND AUDIT**

### 6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

### **FUNCTION**

6.5.1.1 The PORC shall function to advise the Superintendent of Plant-Susquehanna on matters related to nuclear safety as described in Specification 6.5.1.6.

### **COMPOSITION**

6.5.1.2 The PORC shall be composed of the:

Chairman:

Superintendent of Plant-Susquehanna

Member:

Assistant Superintendent - Outages

Member:

Supervisor of Operations

Member:

Technical Supervisor

Member:

Supervisor of Maintenance

Member:

I&C/Computer Supervisor

Member:

Reactor Engineering Supervisor or Unit Reactor Engineer

Member:

Shift Supervisor or Unit Supervisor

Member:

Health Physics Supervisor

Member:

Chemistry Supervisor

#### **ALTERNATES**

6.5.1.3 All alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.

### **MEETING FREQUENCY**

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate.

#### **OUORUM**

6.5.1.5 The quorum of the PORC necessary for the performance of the PORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.



WASHINGTON, D.C. 20555

### PENNSYLVANIA POWER & LIGHT COMPANY

### ALLEGHENY ELECTRIC COOPERATIVE, INC.

**DOCKET NO. 50-388** 

# SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

# AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80 License No. NPF-22

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated April 2, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:
  - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 80 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

This license amendment is effective 30 days after its date of issuance.

FOR THE NUCLEAR PEGULATORY COMMISSION

Walter R. Butler, Director Project Directorate I-2 Division of Reactor Projects - I/II

Attachment: Changes to the Technical Specifications

Date of Issuance: June 26, 1991

# ATTACHMENT TO LICENSE AMENDMENT NO. 80

# FACILITY OPERATING LICENSE NO. NPF-22

### DOCKET NO. 50-388

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages is identified by Amendment number and contain vertical lines indicating the area of change. The overleaf page is provided to maintain document completeness.\*

REMOVE	INSERT
6-7	6-7
6-8	6-8*

# 6.2.3 NUCLEAR SAFETY ASSESSMENT GROUP (NSAG)

### **FUNCTION**

6.2.3.1 The NSAG shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety.

### COMPOSITION

6.2.3.2 The NSAG shall be composed of at least five dedicated, full-time engineers with at least three located onsite, each with a bachelor's degree in engineering or related science and at least two years professional level experience in his field, at least one year of which experience shall be in the nuclear field.

### RESPONSIBILITIES

6.2.3.3 The NSAG shall be responsible for maintaining surveillance of unit activities to provide independent verification\* that these activities are performed correctly and that human errors are reduced as much as practical.

### **AUTHORITY**

6.2.3.4 The NSAG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving unit safety to the Senior Vice President-Nuclear.

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Member:

Supervisor of Maintenance I&C/Computer Supervisor

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Member:

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Member:

Health Physics Supervisor

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Chemistry Supervisor

#### **ALTERNATES**

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#### MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate.

### **OUORUM**

6.5.1.5 The quorum of the PORC necessary for the performance of the PORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.



WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 111 TO FACILITY OPERATING LICENSE NO. NPF-14

AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. NPF-22

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

DOCKET NOS. 50-387 AND 388

### I.O INTRODUCTION

By letter dated April 2, 1990, the Pennsylvania Power and Light Company and Allegheny Electric Cooperative, Inc. (the licensees) submitted a request for changes to the Susquehanna Steam Electric Station, Units 1 and 2, Technical Specification (TS). The requested changes would revise Technical Specifications 6.4 for both Units 1 and 2. The phrase "and Appendix 'A' of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience" is being deleted. Instead, the phrase "except that the licensed operator initial training and requalification programs shall meet or exceed the requirements of 10 CFR Part 55 and utilize the guidance contained in Regulatory Guide 1.8, Rev. 2" is being added.

### 2.0 EVALUATION

With the revision to 10 CFR 55 that became effective on May 26, 1987, portions of Technical Specification 6.4 (Training) became obsolete. In addition, PP&L, in its response to the NRC Notice of Violation dated January 22, 1990, committed to incorporate the guidance of Regulatory Guide 1.8, Rev. 2 into their licensed operator requalification program. This change will delete those references superseded by the revised rulemaking and incorporate those commitments made in the response.

The phrase "and Appendix 'A' of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of the Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience" is being deleted. The basis for the deletion is:

1. Appendix "A" Requalification Program - has been deleted from 10 CFR Part 55. The requalification program is currently addressed under 10 CFR 55.59. PP&L has also committed to utilize the guidance of Regulatory Guide 1.8, Rev. 2 to fulfill the requalification requirement.

9107090052 910626 PDR ADOCK 05000387 PDR PDR 2. Reference to the March 28, 1980 NRC letter (Denton letter) has been deleted. NUREG-1262, "Answers to Questions at Public Meetings Regarding Implementation of 10 CFR 55 on Operator's Licenses," question #385 clearly stated, "the rule (10CFR55) supersedes and should include the requirements of the Harold Denton letter of March 28, 1980." Section A of Enclosure 1 of the Denton letter titled "Eligibility Requirements to be Administered an Examination" has been incorporated into Subpart D of 10 CFR 55 for licensed operators while Section C titled "Requalification Program" has been discussed above.

The response made commitments to the licensed operator requalification program only. The licensee's other licensed operator training programs also meet or exceed 10 CFR 55 and the guidance of Regulatory Guide 1.8, Rev. 2.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: James J. Raleigh

Date: June 26, 1991