



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 27, 2000

Mr. J. A. Scalice
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
CORRECTED PAGES FOR AMENDMENT NOS. 265 AND 256 REGARDING
BORON CREDIT IN SPENT FUEL POOL (TAC NOS. MA9884 AND MA9885)
(TSC 99-17)

Dear Mr. Scalice:

Following the issuance of Amendment No. 265 to Facility Operating License No. DPR-77 and Amendment No. 256 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant (SQN), Units 1 and 2, respectively, your Sequoyah Licensing staff noted minor editorial errors on three pages in each amendment. Bases pages B 3/4 7-12 and B 3/4 7-15 for each unit listed references but the references were not numbered as they should have been. Technical Specification page 5-5a had an inadvertent space in the middle of a sentence. This letter forwards corrected pages for insertion into the amendments. These pages have already been telefaxed to Mr. Zachary Kitts at the Sequoyah site. I apologize for any inconvenience.

Sincerely,

Ronald W. Hernan, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosures: Corrected amendment pages

cc w/enclosures: See next page

December 27, 2000

Mr. J. A. Scalice
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
CORRECTED PAGES FOR AMENDMENT NOS. 265 AND 256 REGARDING
BORON CREDIT IN SPENT FUEL POOL (TAC NOS. MA9884 AND MA9885)
(TSC 99-17)

Dear Mr. Scalice:

Following the issuance of Amendment No. 265 to Facility Operating License No. DPR-77 and Amendment No. 256 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant (SQN), Units 1 and 2, respectively, your Sequoyah Licensing staff noted minor editorial errors on three pages in each amendment. Bases pages B 3/4 7-12 and B 3/4 7-15 for each unit listed references but the references were not numbered as they should have been. Technical Specification page 5-5a had an inadvertent space in the middle of a sentence. This letter forwards corrected pages for insertion into the amendments. These pages have already been telefaxed to Mr. Zachary Kitts at the Sequoyah site. I apologize for any inconvenience.

Sincerely,

/RA/
Ronald W. Hernan, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosures: Corrected amendment pages
cc w/enclosures: See next page

DISTRIBUTION (w/enclosures):

PUBLIC
WBeckner (e-mail)
RidsNrrDlpmLpdii2
JWermiel
RidsAcrsAcnwMailCenter
GHill (4 Hardcopies)
RidsNrrDlpmLpdii
RidsRgn2MailCenter
RidsOgcRp
PDII-2\Reading
LKopp (e-mail)
RHernan (Hardcopy)
BClayton (Hardcopy)

DOCUMENT NAME: G:\PDII-2\Sequoyah\AMDA9884 corrected pages.wpd

OFFICE	PDII-2/PM	PDII-2/LA	PDII-2/SC
NAME	RHernan <i>RWH</i>	BClayton <i>BC</i>	RCorreia <i>RWH</i>
DATE	12/27/00	12/27/00	12/27/00

OFFICIAL RECORD COPY

PLANT SYSTEMS

BASES (continued)

REFERENCES

1. Stanley E. Turner (Holtec International), "Criticality Safety Analyses of Sequoyah Spent Fuel Racks with Alternative Arrangements," HI-992349
2. B. K. Grimes (NRC GL78011), "OT Position for Review and Acceptance of Spent Fuel Storage and Handling Applications", April 14, 1978
3. L. Kopp, "Guidance On The Regulatory Requirements For Criticality Analysis Of Fuel Storage At Light-Water Reactor Power Plants", August 19, 1998
4. UFSAR, Section 4.3.2.7, "Criticality of Fuel Assemblies"
5. Double contingency principle of ANSI N16.1-1975, as specified in the April 14, 1978 NRC letter (Section 1.2) and implied in the proposed revision to Regulatory Guide 1.13 (Section 1.4, Appendix A).
6. K. K. Niyogi (Holtec International), "Boron Dilution Analysis," HI-992302
7. FSAR, Section 15.4.5
8. NRC letter to TVA dated August 1, 1990, " Increase Fuel Enrichment to 5.0 Weight Percent (TAC Nos. 76074, 76075, 76774, 76775) (TS 90-12) - Sequoyah Nuclear Plant, Units 1 and 2"

PLANT SYSTEMS

BASES (continued)

REFERENCES

1. Stanley E. Turner (Holtec International), "Criticality Safety Analyses of Sequoyah Spent Fuel Racks with Alternative Arrangements," HI-992349
2. B. K. Grimes (NRC GL78011), "OT Position for Review and Acceptance of Spent Fuel Storage and Handling Applications", April 14, 1978
3. L. Kopp, "Guidance On The Regulatory Requirements For Criticality Analysis Of Fuel Storage At Light-Water Reactor Power Plants", August 19, 1998
4. UFSAR, Section 4.3.2.7, "Criticality of Fuel Assemblies"
5. Double contingency principle of ANSI N16.1-1975, as specified in the April 14, 1978 NRC letter (Section 1.2) and implied in the proposed revision to Regulatory Guide 1.13 (Section 1.4, Appendix A).
6. K. K. Niyogi (Holtec International), "Boron Dilution Analysis," HI-992302
7. FSAR, Section 15.4.5
8. NRC letter to TVA dated August 1, 1990, " Increase Fuel Enrichment to 5.0 Weight Percent (TAC Nos. 76074, 76075, 76774, 76775) (TS 90-12) - Sequoyah Nuclear Plant, Units 1 and 2"

PLANT SYSTEMS

BASES (continued)

REFERENCES

1. NRC letter to TVA dated April 28, 1993, "Issuance of Amendments (TAC Nos. M83068 and M83069)"
2. Stanley E. Turner (Holtec International), "Criticality Safety Analyses of Sequoyah Spent Fuel Racks with Alternative Arrangements," HI-992349
3. FSAR, Section 15.4.5
4. K. K. Niyogi (Holtec International), "Boron Dilution Analysis," HI-992302

PLANT SYSTEMS

BASES (continued)

REFERENCES

1. NRC letter to TVA dated April 28, 1993, "Issuance of Amendments (TAC Nos. M83068 and M83069)"
2. Stanley E. Turner (Holtec International), "Criticality Safety Analyses of Sequoyah Spent Fuel Racks with Alternative Arrangements," HI-992349
3. FSAR, Section 15.4.5
4. K. K. Niyogi (Holtec International), "Boron Dilution Analysis," HI-992302

5.6 FUEL STORAGE

2. Region 2 is designed to accommodate fuel of 4.95 ± 0.05 wt% U-235 initial enrichment burned to at least 30.27 MWD/KgU (assembly average), or fuel of other enrichment with a burnup yielding an equivalent reactivity in the fuel racks. The minimum required assembly average burnup in MWD/KgU and cooling time is given by the equations in Table 5.6-3 in terms of E, where E is the initial enrichment in the axial zone of highest enrichment (wt% U-235). The minimum required burnups are illustrated in Figure 5.6-3 in terms of the initial enrichment and cooling time.

Restrictions in Region 2

The following restrictions apply to the storage of spent fuel in the Region 2 cells:

- 1) The spent fuel shall conform to the minimum burnup requirements defined by the equations in Table 5.6-3. Linear interpolation between cooling times may be made if desired.
- 2) For the interface with Region 1 storage cells, fresh fuel in Region 1 shall not be stored adjacent to spent fuel assemblies in the Region 2 storage cells.
3. Region 3 is designed to accommodate fuel of 4.95 ± 0.05 wt% U-235 initial enrichment (or fuel assemblies of any lower reactivity) in a 2-out-of-4 checkerboard arrangement with water-filled cells. The water-filled cells shall not contain any components bearing any fissile material, but may accommodate miscellaneous items or equipment.

Restrictions in Region 3

- 1) For the interface between Region 1 and Region 3 storage regions, fresh fuel assemblies shall not be stored adjacent to each other.
- 2) If miscellaneous items or equipment are stored in the water cells of Region 3, the total volume of the miscellaneous items shall be no more than 75% of the storage cell volume.
- 3) No fuel rods, assemblies, or items containing fissile material shall be stored in the water cells of Region 3.

An empty cell is less reactive than any cell containing fuel and therefore may be used as a Region 1, Region 2, or Region 3 cell in any arrangement.

- d. Region 2 array described above may be used in the 15 x 15 storage rack module in the cask loading area of the cask pit.
- e. A nominal concentration of 2000 ppm boron in the pool water. This concentration of soluble boron provides a margin sufficient to allow timely detection of a boron dilution accident and corrective action before the minimum concentration (700 ppm) required to protect against the most severe postulated fuel handling accident or before the minimum concentration (300 ppm) required to maintain the storage configuration design basis (k_{eff} less than 0.95) is reached.

April 28, 1993

5.6 FUEL STORAGE

2. Region 2 is designed to accommodate fuel of 4.95 ± 0.05 wt% U-235 initial enrichment burned to at least 30.27 MWD/KgU (assembly average), or fuel of other enrichments with a burnup yielding an equivalent reactivity in the fuel racks. The minimum required assembly average burnup in MWD/KgU and cooling time is given by the equations in Table 5.6-3 in terms of E, where E is the initial enrichment in the axial zone of highest enrichment (wt% U-235). The minimum required burnups are illustrated in Figure 5.6-3 in terms of the initial enrichment and cooling time.

Restrictions in Region 2

The following restrictions apply to the storage of spent fuel in the Region 2 cells:

- 1) The spent fuel shall conform to the minimum burnup requirements defined by the equations in Table 5.6-3. Linear interpolation between cooling times may be made if desired.
- 2) For the interface with Region 1 storage cells, fresh fuel in Region 1 shall not be stored adjacent to spent fuel assemblies in the Region 2 storage cells.
3. Region 3 is designed to accommodate fuel of 4.95 ± 0.05 wt% U-235 initial enrichment (or fuel assemblies of any lower reactivity) in a 2-out-of-4 checkerboard arrangement with water-filled cells. The water-filled cells shall not contain any components bearing any fissile material, but may accommodate miscellaneous items or equipment.

Restrictions in Region 3

- 1) For the interface between Region 1 and Region 3 storage regions, fresh fuel assemblies shall not be stored adjacent to each other.
- 2) If miscellaneous items or equipment are stored in the water cells of Region 3, the total volume of the miscellaneous items shall be no more than 75% of the storage cell volume.
- 3) No fuel rods, assemblies, or items containing fissile material shall be stored in the water cells of Region 3.

An empty cell is less reactive than any cell containing fuel and therefore may be used as a Region 1, Region 2, or Region 3 cell in any arrangement.

- d. Region 2 array described above may be used in the 15 x 15 storage rack module in the cask loading area of the cask pit.
- e. A nominal concentration of 2000 ppm boron in the pool water. This concentration of soluble boron provides a margin sufficient to allow timely detection of a boron dilution accident and corrective action before the minimum concentration (700 ppm) required to protect against the most severe postulated fuel handling accident or before the minimum concentration (300 ppm) required to maintain the storage configuration design basis (k_{eff} less than 0.95) is reached.

Mr. J. A. Scalice
Tennessee Valley Authority

cc:

Mr. Karl W. Singer, Senior Vice President
Nuclear Operations
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Jack A. Bailey
Vice President
Engineering & Technical Services
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Richard T. Purcell
Site Vice President
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

General Counsel
Tennessee Valley Authority
ET 10H
400 West Summit Hill Drive
Knoxville, TN 37902

Mr. Robert J. Adney, General Manager
Nuclear Assurance
Tennessee Valley Authority
5M Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Mark J. Burzynski, Manager
Nuclear Licensing
Tennessee Valley Authority
4X Blue Ridge
1101 Market Street
Chattanooga, TN 37402-2801

SEQUOYAH NUCLEAR PLANT

Mr. Pedro Salas, Manager
Licensing and Industry Affairs
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

Mr. D. L. Koehl, Plant Manager
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

Mr. Russell A. Gibbs
Senior Resident Inspector
Sequoyah Nuclear Plant
U.S. Nuclear Regulatory Commission
2600 Igou Ferry Road
Soddy Daisy, TN 37379

Mr. Lawrence E. Nanney, Director
Division of Radiological Health
Dept. of Environment & Conservation
Third Floor, L and C Annex
401 Church Street
Nashville, TN 37243-1532

County Executive
Hamilton County Courthouse
Chattanooga, TN 37402-2801

Ms. Ann Harris
305 Pickel Road
Ten Mile, TN 37880