

May 30, 1989

Docket Nos. 50-387/388

Mr. Harold W. Keiser
Senior Vice President-Nuclear
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

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ACRS (10)

Dear Mr. Keiser:

SUBJECT: REQUEST FOR EXEMPTION FROM 10 CFR 20.103(c)(2)
(TAC NOS. 73119/73120)

RE: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

We have received your January 17, 1989 request for an exemption from the requirement of 10 CFR 20.103(c)(2) related to a 12 month time interval for physical examinations to assure that an individual is physically able to use the respiratory protective equipment in an environment containing airborne radioactive material. Specifically, you have requested an exemption to permit the physical examinations once "every 9 to 15 months; provided that the total time over any three consecutive physical examination periods does not exceed 39 months."

Our review indicates that your request is consistent with the staff's position on implementation of the time interval requirements of 10 CFR 20.103(c)(2), and an exemption from 10 CFR 20.103(c)(2) is not needed. However, if you still wish to pursue your January 17, 1989 request for an exemption, let us know your decision at your earliest convenience so that we can process your request in a timely fashion.

Sincerely,

/s/

Mohan C. Thadani, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

cc: See next page

[KEISER/LET]

PDI-2/LA
MO'Brien
5/23/89

PDI-2/PM
MThadani:tr
5/23/89

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5/26/89

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, reading "Mohan C. Thadani", is positioned above the typed name and title.

Mohan C. Thadani, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

cc: See next page

Mr. Harold W. Keiser
Pennsylvania Power & Light Company

Susquehanna Steam Electric Station
Units 1 & 2

cc:

Jay Silberg, Esq.
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2300 N Street N.W.
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Bryan A. Snapp, Esq.
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Licensing Group Supervisor
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Mr. S. Barber
Resident Inspector
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Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
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Mr. Jesse C. Tilton, III
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Mr. W. H. Hirst, Manager
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Atlantic Electric
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1199 Black Horse Pike
Pleasantville, New Jersey 08232

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

May 22, 1989

Docket No. 50-387

Mr. Harold W. Keiser
Senior Vice President-Nuclear
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Keiser:

SUBJECT: TECHNICAL SPECIFICATION CHANGES SUPPORTING ELIMINATION OF THE STEAM
CONDENSING MODE OF THE RESIDUAL HEAT REMOVAL (RHR) SYSTEM OPERATION
(TAC NO. 72851)

RE: SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station (SSES), Unit 1. This amendment is in response to your letter dated February 24, 1989.

This amendment revises the SSES Unit 1 Technical Specifications supporting modifications to eliminate the steam condensing mode of the RHR system operation.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 91 to License No. NPF-14
2. Safety Evaluation

cc w/enclosures:
See next page

[KEISER LTR]

PDI-2/D
WButler
5/12/89

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script, reading "Walter R. Butler", is positioned above the typed name and title.

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

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License No. NPF-14
2. Safety Evaluation

cc w/enclosures:
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Mr. Harold W. Keiser
Pennsylvania Power & Light Company

Susquehanna Steam Electric Station
Units 1 & 2

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Regional Administrator, Region I
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King of Prussia, Pennsylvania 19406



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated February 24, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 91 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented prior to startup, following the Unit 1 fourth refueling and inspection outage, expected to occur on June 2, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **May 22, 1989**

PDI-2/DA
MThadani
5/25/89

ML
PDI-2/PM
MThadani:tr
5/2/89

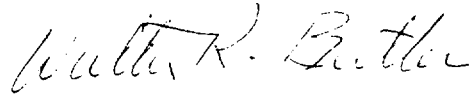
OGC APH
5/4/89

PDI-2/D
WButler
5/10/89

WB

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FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **May 22, 1989**

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf pages are provided to maintain document completeness.*

REMOVE

3/4 6-21
3/4 6-22*

3/4 6-23*
3/4 6-24

3/4 8-29*
3/4 8-30

INSERT

3/4 6-21
3/4 6-22*

3/4 6-23*
3/4 6-24

3/4 8-29*
3/4 8-30

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>ISOLATION SIGNAL(s)^(a)</u>
<u>Automatic Isolation Valves (Continued)</u>		
<u>Containment Atmosphere Sample</u>		
SV-15734 A,B	N/A	B,Y
SV-15736 A	N/A	B,Y
SV-15736 B	N/A	B,Y
SV-15740 A,B	N/A	B,Y
SV-15742 A,B	N/A	B,Y
SV-15750 A,B	N/A	B,Y
SV-15752 A,B	N/A	B,Y
SV-15774 A,B	N/A	B,Y
SV-15776 A	N/A	B,Y
SV-15776 B	N/A	B,Y
SV-15780 A,B	N/A	B,Y
SV-15782 A,B	N/A	B,Y
<u>Nitrogen Makeup</u>		
SV-15737	N/A	B,Y,R
SV-15738	N/A	B,Y,R
SV-15767	N/A	B,Y,R
SV-15789	N/A	B,Y,R
<u>Reactor Coolant Sample</u>		
HV-143F019	2	B,C
HV-143F020	2	B,C
<u>Liquid Radwaste</u>		
HV-16108 A1,A2	15	B,Z
HV-16116 A1,A2	15	B,Z
<u>RHR - Suppression Pool</u>		
<u>Cooling/Spray^(c)</u>		
HV-151F028 A,B	90	X,Z
<u>CS Test^{(b)(c)}</u>		
HV-152F015 A,B	60	X,Z
<u>HPCI Suction^{(b)(c)}</u>		
HV-155F042	90	L, LB

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>ISOLATION SIGNAL(s)^(a)</u>
<u>Automatic Isolation Valves (Continued)</u>		
<u>Suppression Pool Cleanup^(b)</u>		
HV-15766	30	A,Z
HV-15768	30	A,Z
<u>HPCI Vacuum Breaker</u>		
HV-155F075	15	LB,Z
HV-155F079	15	LB,Z
<u>RCIC Vacuum Breaker</u>		
HV-149F062	10	KB,Z
HV-149F084	10	KB,Z
<u>TIP Ball Valves^(d)</u>		
C51-J004 A,B,C,D,E	5	A,Z
b. <u>Manual Isolation Valves</u>		
<u>MSIV-LCS Bleed Valve</u>		
HV-139F001 B,F,K,P		
<u>Feedwater^(e)</u>		
HV-141F032 A,B		
<u>RWCU Return</u>		
HV-14182 A,B		
<u>RCIC Injection</u>		
HV-149F013		
1-49-020		

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Manual Isolation Valves (Continued)

RCIC Suction^{(b)(c)}

HV-149F031

RCIC Turbine Exhaust^(b)

HV-149F059

RCIC Vacuum Pump Discharge^(b)

HV-149F060

HPCI Injection

HV-155F006

1-55-038

RHR - Shutdown Cooling Return/

LPCI Injection

HV-151F015 A,B

RHR - Suppression Pool Suction^{(b)(c)}

HV-151F004 A,B,C,D

RHR Heat Exchanger Vent^(c)

HV-151F103 A,B

CS Injection

HV-152F005 A,B

HV-152F037 A,B

CS Suction^{(b)(c)}

HV-152F001 A,B

Containment Instrument Gas

SV-12654 A,B

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Manual Isolation Valves (Continued)

SLCS^(b)

HV-148F006

Demineralized Water

1-41-017

1-41-018

ILRT

1-57-193

1-57-194

HPCI Turbine Exhaust^(b)

HV-155F066

RHR - Shutdown Cooling Return/

LPCI Injection - Pressure Equalizing Valve

HV-151F122 A,B

c. Other Valves

Feedwater

141F010 A,B

RHR - Shutdown Cooling Suction

PSV-151F126

RHR - Shutdown Cooling Return/

LPCI Injection

HV-151F050 A,B

RHR-Suppression Pool

Cooling/Spray^(c)

HV-151F011 A,B

TABLE 3.8.4.2.1-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - CONTINUOUS

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-01222A	RHRSW
HV-01222B	RHRSW
HV-01224A1	RHRSW
HV-01224B1	RHRSW
HV-01224A2	RHRSW
HV-01224B2	RHRSW
*HV-01112A	ESW
*HV-01112B	ESW
*HV-01122A	ESW
*HV-01122B	ESW
*HV-01112C	ESW
*HV-01112D	ESW
*HV-01122C	ESW
*HV-01122D	ESW
*HV-01110A	ESW
*HV-01110B	ESW
*HV-01120A	ESW
*HV-01120B	ESW
*HV-01110C	ESW
*HV-01110D	ESW
*HV-01120C	ESW
*HV-01120D	ESW
*HV-01110E	ESW
*HV-01120E	ESW
*HV-01112E	ESW
*HV-01122E	ESW
HV-08693A	ESW
HV-08693B	ESW
HV-01201A1	RHRSW
HV-01201A2	RHRSW
HV-01201B1	RHRSW
HV-01201B2	RHRSW
HV-11210A	RHRSW
HV-11210B	RHRSW
HV-11215A	RHRSW
HV-11215B	RHRSW
HV-15766	Cont. Isol.
HV-15768	Cont. Isol.
HV-12603	Cont. Isol.
HV-11345	Cont. Isol.
HV-11313	Cont. Isol.
HV-11346	Cont. Isol.
HV-11314	Cont. Isol.
HV-E11-1F009	RHR

* Continuous bypass not required when corresponding diesel generator is not aligned to the Class 1E distribution system.

TABLE 3.8.4.2.1-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION CONTINUOUS

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-E11-1F040	RHR
HV-G33-1F001	RWCU
HV-E11-1F103A	RHR
HV-E11-1F075A	RHRSW
HV-E11-1F048A	RHR
HV-E11-1F006C	RHR
HV-E11-1F004C	RHR
HV-E11-1F015A	RHR
HV-E11-1F024A	RHR
HV-E21-1F015A	CS
HV-E41-1F002	HPCI
HV-B21-1F016	NSSS
HV-E11-1F022	RHR
HV-E11-1F010A	RHR
HV-E11-1F004A	RHR
HV-E11-1F006A	RHR
HV-E11-1F027A	RHR
HV-E11-1F007A	RHR
HV-E11-1F104A	RHR
HV-E11-1F028A	RHR
HV-E11-1F047A	RHR
HV-E11-1F073A	RHRSW
HV-E11-1F003A	RHR
HV-E11-1F017A	RHR
HV-E21-1F001A	CS
HV-E21-1F031A	CS
HV-E21-1F004A	CS
HV-E21-1F005A	CS
HV-E11-1F021A	RHR
HV-E11-1F016A	RHR
HV-15112	RHR
HV-E51-1F007	RCIC
HV-E51-1F084	RCIC
HV-E11-1F027B	RHR
HV-E11-1F048B	RHR
HV-E11-1F015B	RHR
HV-E11-1F006B	RHR
HV-E11-1F021B	RHR
HV-E11-1F010B	RHR
HV-E11-1F004B	RHR
HV-E11-1F007B	RHR
HV-E11-1F104B	RHR



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. NPF-14
PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM-ELECTRIC STATION, UNIT 1

1.0 INTRODUCTION

By letter dated February 24, 1989, Pennsylvania Power & Light Company requested an amendment to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station (SSES), Unit 1. The proposed amendment would revise the Technical Specifications in support of the proposed modification which eliminates the Steam Condensing Mode of the Residual Heat Removal (RHR) System operation. Similar changes were approved for SSES Unit 2 operation in Amendment No. 49, dated May 24, 1988.

2.0 EVALUATION

The licensee states that the steam condensing mode of the RHR system operation is not a safety related function. The steam condensing mode will be eliminated by converting the valves, F011 A/B, to locked closed manual valves by removing the electrical connections, controls, and position indicating lights. The computer points for the position will be deleted. These changes are acceptable because the steam condensing mode of the RHR is not an essential mode (unlike LPCI, containment cooling, and shutdown cooling modes). Moreover, SSES, Unit 1 does not use the steam condensing mode of operation of RHR for mitigation of transients and accidents. For these reasons, the staff concludes that the licensee's proposal to suspend the steam condensing mode of the RHR operation and the associated changes to the Technical Specifications are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously

issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (54 FR 15834) on April 19, 1989 and consulted with the State of Pennsylvania. No public comments were received, and the State of Pennsylvania did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Mohan Thadani

Dated: May 22, 1989