

August 16, 1989

Docket Nos. 50-387/388

Mr. Harold W. Keiser
Senior Vice President-Nuclear
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Keiser:

DISTRIBUTION

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SUBJECT: TECHNICAL SPECIFICATION CHANGES TO REFLECT APPENDIX R MODIFICATIONS TO EMERGENCY SERVICE WATER VALVES (TAC NOS. 73038/73039)

RE: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

The Commission has issued the enclosed Amendment No. 92 to Facility Operating License No. NPF-14 and Amendment No. 56 to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2. These amendments are in response to your letter dated April 14, 1989.

These amendments revise the Technical Specifications to reflect modifications to the Emergency Service Water system valves which provide cooling water to the diesel generators. The modifications are being made as a result of compliance with 10 CFR Part 50, Appendix R, Part III.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 92 to License No. NPF-14
2. Amendment No. 56 to License No. NPF-22
3. Safety Evaluation

cc w/enclosures:
See next page

[TAC NOS. 73038/73039]

PDI-2/LA
MO'Brien
7/11/89

PDI-2/PM
MThadani:mj
6/27/89

SELB
SPLB/C
~~J. Craig F. Rose~~
7/11/89

PDI-2/D
WButler
8/15/89

OGC
Bamb
8/18/89

JFol
11

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PDR ADOCK 05000387
P PNU

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 16, 1989

Docket Nos. 50-387/388

Mr. Harold W. Keiser
Senior Vice President-Nuclear
Pennsylvania Power and Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Keiser:

SUBJECT: TECHNICAL SPECIFICATION CHANGES TO REFLECT APPENDIX R MODIFICATIONS
TO EMERGENCY SERVICE WATER VALVES (TAC NOS. 73038/73039)

RE: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

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A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script that reads "Walter R. Butler".

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 92 to License No. NPF-14
2. Amendment No. 56 to License No. NPF-22
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Harold W. Keiser
Pennsylvania Power & Light Company

Susquehanna Steam Electric Station
Units 1 & 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated April 14, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 92 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

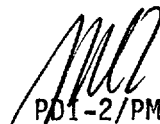
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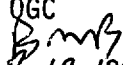
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II


Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 16, 1989


PDI-2/LR
M O'Brien
7/11/89

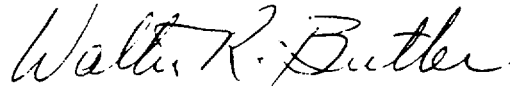

PDI-2/PM
MThadani:mj
7/11/89

OGC

8/18/89

PDI-2/D
WButler

8/15/89

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 16, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf page(s) are provided to maintain document completeness.*

REMOVE

3/4 8-13
3/4 8-14

3/4 8-29
3/4 8-30

3/4 8-32a
3/4 8-32b

INSERT

3/4 8-13
3/4 8-14*

3/4 8-29
3/4 8-30*

3/4 8-32a*
3/4 8-32b

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- 5) Channel "A" battery 2D612:
323 amperes for 60 seconds
96 amperes for the remainder of the 4 hour test.
 - 6) Channel "B" battery 2D622:
324 amperes for 60 seconds
96 amperes for the remainder of the 4 hour test.
 - 7) Channel "C" battery 2D632:
297 amperes for 60 seconds
80 amperes for the remainder of the 4 hour test.
 - 8) Channel "D" battery 2D642:
300 amperes for 60 seconds
83 amperes for the remainder of the 4 hour test.
 - 9) Channel "H" battery OD595:
253 amperes for the first 60 seconds
75 amperes for the remainder of the 4 hour test.
- c) For 250-volt batteries:
- 1) Battery bank 1D650:
1120 amperes for 60.0 seconds
599 amperes for 29.0 minutes
99 amperes for 120.0 minutes
27 amperes for 90.0 minutes
 - 2) Battery bank 1D660:
887 amperes for 60.0 seconds
396 amperes for 9.0 minutes
366 amperes for 20.0 minutes
325 amperes for 90.0 minutes
187 amperes for 119.0 minutes
229 amperes for 60.0 seconds
- e. At least once per 60 months by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. Once per 60 month interval, this performance discharge test may be performed in lieu of the battery service test.
- f. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

TABLE 4.8.2.1-1

BATTERY SURVEILLANCE REQUIREMENTS

Parameter	CATEGORY A ⁽¹⁾		CATEGORY B ⁽²⁾	
	Limits for each designated pilot cell	Limits for each connected cell	Limits for each connected cell	Allowable ⁽³⁾ value for each connected cell
Electrolyte Level	>Minimum level indication mark, and < 1/4" above maximum level indication mark	>Minimum level indication mark, and < 1/4" above maximum level indication mark	>Minimum level indication mark, and < 1/4" above maximum level indication mark	Above top of plates, and not overflowing
Float Voltage	> 2.13 volts	≥ 2.13 volts ^(c)	≥ 2.13 volts ^(c)	> 2.07 volts
Specific Gravity ^(a)	≥ 1.200 ^(b)	≥ 1.195 ^(b) . Average of all connected cells > 1.205 ^(b)	≥ 1.195 ^(b) . Average of all connected cells > 1.205 ^(b)	Not more than .020 below the average of all connected cells Average of all connected cells ≥ 1.195 ^(b)

- (a) Corrected for electrolyte temperature and level.
- (b) Or battery charging current is less than 0.01, 0.1 and 0.25 amperes for the 24, 125 and 250 volt batteries, respectively, when on float charge.
- (c) May be corrected for average electrolyte temperature.
- (1) For any Category A parameter(s) outside the limit(s) shown, the battery may be considered OPERABLE provided that within 24 hours all the Category B measurements are taken and found to be within their allowable values, and provided all Category A and B parameter(s) are restored to within limits within the next 8 days.
- (2) For any Category B parameter(s) outside the limit(s) shown, the battery may be considered OPERABLE provided that the Category B parameters are within their allowable values and provided the Category B parameter(s) are restored to within limits within 7 days.
- (3) Any Category B parameter not within its allowable value indicates an inoperable battery.

TABLE 3.8.4.2.1-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - CONTINUOUS

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-01222A	RHRSW
HV-01222B	RHRSW
HV-01224A1	RHRSW
HV-01224B1	RHRSW
HV-01224A2	RHRSW
HV-01224B2	RHRSW
HV-08693A	ESW
HV-08693B	ESW
HV-01201A1	RHRSW
HV-01201A2	RHRSW
HV-01201B1	RHRSW
HV-01201B2	RHRSW
HV-11210A	RHRSW
HV-11210B	RHRSW
HV-11215A	RHRSW
HV-11215B	RHRSW
HV-15766	Cont. Isol.
HV-15768	Cont. Isol.
HV-12603	Cont. Isol.
HV-11345	Cont. Isol.
HV-11313	Cont. Isol.
HV-11346	Cont. Isol.
HV-11314	Cont. Isol.
HV-E11-1F009	RHR

TABLE 3.8.4.2.1-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION CONTINUOUS

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-E11-1F040	RHR
HV-G33-1F001	RWCU
HV-E11-1F103A	RHR
HV-E11-1F075A	RHRSW
HV-E11-1F048A	RHR
HV-E11-1F006C	RHR
HV-E11-1F004C	RHR
HV-E11-1F015A	RHR
HV-E11-1F024A	RHR
HV-E21-1F015A	RHR
HV-E41-1F002	CS
HV-821-1F016	HPCI
HV-E11-1F022	NSSS
HV-E11-1F010A	RHR
HV-E11-1F004A	RHR
HV-E11-1F006A	RHR
HV-E11-1F027A	RHR
HV-E11-1F007A	RHR
HV-E11-1F104A	RHR
HV-E11-1F028A	RHR
HV-E11-1F047A	RHR
HV-E11-1F073A	RHR
HV-E11-1F003A	RHR
HV-E11-1F017A	RHR
HV-E21-1F001A	RHR
HV-E21-1F031A	RHR
HV-E21-1F004A	RHR
HV-E21-1F005A	RHR
HV-E11-1F021A	RHR
HV-E11-1F016A	RHR
HV-15112	RHR
HV-E51-1F007	RHR
HV-E51-1F084	RHR
HV-E11-1F027B	RCIC
HV-E11-1F048B	RCIC
HV-E11-1F015B	RHR
HV-E11-1F006B	RHR
HV-E11-1F021B	RHR
HV-E11-1F010B	RHR
HV-E11-1F004B	RHR
HV-E11-1F007B	RHR
HV-E11-1F104B	RHR

ELECTRICAL POWER SYSTEMS

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - AUTOMATIC

LIMITING CONDITION FOR OPERATION

3.8.4.2.2 The thermal overload protection of each valve shown in Table 3.8.4.2.2-1 shall be bypassed automatically by an OPERABLE bypass device integral with the motor starter.

APPLICABILITY: When diesel generator E is not aligned to the Class 1E distribution system.

ACTION:

- a. With thermal overload protection automatic bypass inoperable for one or more valves listed above, take administrative action to continuously bypass the thermal overload within 8 hours, or verify that all diesel generator E ESW valves are closed and diesel generator E is not running within 8 hours.

SURVEILLANCE REQUIREMENTS

4.8.4.2.2.1 The automatic bypass of thermal overload protection for those valves listed above shall be demonstrated OPERABLE at least once per 18 months.

TABLE 3.8.4.2.2-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - AUTOMATIC

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-01110E	ESW
HV-01120E	ESW
HV-01112E	ESW
HV-01122E	ESW



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-388

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. NPF-22

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by the Pennsylvania Power & Light Company, dated April 14, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 56 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 16, 1989

[Signature]
PDI-2/LA
MO/Brien
7/11/89

[Signature]
PDI-2/PM
MThadani:mj
7/11/89

OGC
[Signature]
8/3/89

[Signature]
PDI-2/D
WButler
8/15/89

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 16, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following pages of the Appendix A Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The overleaf pages are provided to maintain document completeness.*

REMOVE

3/4 8-13
3/4 8-13a

3/4 8-31
3/4 8-32

3/4 8-34a
3/4 8-34b

INSERT

3/4 8-13*
3/4 8-13a

3/4 8-31
3/4 8-32*

3/4 8-34a*
3/4 8-34b

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months by verifying that:
1. The cells, cell plates, and battery racks show no visual indication of physical damage or abnormal deterioration,
 2. The cell-to-cell and terminal connections are clean, tight, free of corrosion, and coated with anticorrosion material,
 3. The resistance of each cell-to-cell and terminal connection of each 125-volt and 250-volt battery is less than or equal to 150×10^{-6} ohm, and
 4. The battery charger, for at least 4 hours, will supply at least:
 - a) For the \pm 24-volt batteries, 25 amperes at a minimum of 25.7 volts.
 - b) For the 125-volt batteries, 100 amperes at a minimum of 127.8 volts.
 - c) For the 250-volt batteries, 300 amperes at a minimum of 255.6 volts.
 - d) For the 125 volt generator E batteries, 200 amperes at a minimum of 127.8 volts
- d. At least once per 18 months by verifying that either:
1. The battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for the design duty cycle when the battery is subjected to a battery service test, or
 2. The battery capacity is adequate to supply a dummy load of the following profile, which is verified to be greater than the actual emergency loads, while maintaining the battery terminal voltage greater than or equal to \pm 21, 105 or 210 volts, as applicable.
 - a) For \pm 24-volt battery banks 2D670, 2D670-1, 2D680, and 2D680-1, 9.37 amperes for the entire 4-hour test.
 - b) For 125-volt batteries:
 - 1) Channel "A" battery 1D612: 325 amperes for 60 seconds
95 amperes for the remainder of the 4 hour test
 - 2) Channel "B" battery 1D622: 325 amperes for 60 seconds
95 amperes for the remainder of the 4 hour test
 - 3) Channel "C" battery 1D632: 294 amperes for 60 seconds
73 amperes for the remainder of the 4 hour test
 - 4) Channel "D" battery 1D642: 297 amperes for 60 seconds
76 amperes for the remainder of the 4 hour test.
 - 5) Channel "A" battery 2D612: 323 amperes for 60 seconds
96 amperes for the remainder of the 4 hour test
 - 6) Channel "B" battery 2D622: 324 amperes for 60 seconds
96 amperes for the remainder of the 4 hour test

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- 7) Channel "C" battery 2D632: 297 amperes for 60 seconds
80 amperes for the remainder of the 4 hour test
 - 8) Channel "D" battery 2D642: 300 amperes for 60 seconds
83 amperes for the remainder of the 4 hour test
 - 9) Channel "H" battery 0D595: 253 amperes for the first
60 seconds, 75 amperes for the remainder of the 4
hour test.
- c) For 250-volt batteries:
- 1) Battery bank 2D650:
458 amperes for 60 seconds
251 amperes for 239 minutes
 - 2) Battery bank 2D660:
1119 amperes for 60 seconds
244 amperes for 239 minutes
- e. At least once per 60 months by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. Once per 60-month interval, this performance discharge test may be performed in lieu of the battery service test.
- f. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

TABLE 3.8.4.2.1-1

MOTOR-OPERATED VALVES THERMAL OVERLOAD PROTECTION CONTINUOUS

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
HV-01222A	RHRSW
HV-01222B	RHRSW
HV-01224A1	RHRSW
HV-01224B1	RHRSW
HV-01224A2	RHRSW
HV-01224B2	RHRSW
HV-21144A	ESW
HV-21144B	ESW
HV-08693A	ESW
HV-08693B	ESW
HV-01201A1	RHRSW
HV-01201A2	RHRSW
HV-01201B1	RHRSW
HV-01201B2	RHRSW
HV-21210A	RHRSW
HV-21210B	RHRSW
HV-21215A	RHRSW
HV-21215B	RHRSW
HV-25766	Cont. Isol.
HV-25768	Cont. Isol.
HV-22603	Cont. Isol.

ELECTRICAL POWER SYSTEMS

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - AUTOMATIC

LIMITING CONDITION FOR OPERATION

3.8.4.2.2 The thermal overload protection of each valve shown in Table 3.8.4.2.2-1 shall be bypassed automatically by an OPERABLE bypass device integral with the motor starter.

APPLICABILITY: When diesel generator E is not aligned to the Class 1E distribution system.

ACTION:

- a. With thermal overload protection automatic bypass inoperable for one or more valves listed above, take administrative action to continuously bypass the thermal overload within 8 hours, or verify that all diesel generator E ESW valves are closed and diesel generator E is not running within 8 hours.

SURVEILLANCE REQUIREMENTS

4.8.4.2.2.1 The automatic bypass of thermal overload protection for those valves listed above shall be demonstrated OPERABLE at least once per 18 months.

TABLE 3.8.4.2.2-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION - AUTOMATIC

<u>Valve Number</u>	<u>System(s) Affected</u>
HV-01110E	ESW
HV-01120E	ESW
HV-01112E	ESW
HV-01122E	ESW



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. NPF-14 AND

AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. NPF-22

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NOS. 50-387 AND 50-388

SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2

1.0 INTRODUCTION

By letter dated April 14, 1989, Pennsylvania Power & Light Company requested amendments to Facility Operating License Nos. NPF-14 and NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2. The proposed amendments would change the Technical Specifications to delete the Emergency Service Water (ESW) valves associated with diesel generators "A" through "E" from thermal overload protection requirements, add ESW motor operated valves HV-01112E and HV-01122E associated with diesel generator "E" to the thermal overload protection, and revise the load profile for channel "H" battery OD595.

2.0 EVALUATION

As a result of evaluations in accordance with 10 CFR Part 50, Appendix R, the licensee has proposed to modify ESW system. The modifications involve removing the auto-loop transfer for diesel generators "A" through "E", adding auto-closure to loop A ESW valves for diesel generator "E", and revision to the load profile for the diesel generator "E" 125 v dc battery. The licensee states that as a result of removal of auto-loop transfer function, the diesel generators "A" through "E" ESW valves no longer have a safety related function other than maintaining the flowpath integrity or an isolation boundary when diesel generators are not aligned. The licensee also states that thermal overload protection for these valves need neither be continuously bypassed nor does it have the capability of being bypassed. Therefore, the licensee proposes to remove the thermal overload protection bypass for the affected valves. The staff finds that the removal of the ESW valves associated with the diesel generators "A" through "E" from thermal overload protection requirements of Table 3.8.6.2.1-1 is acceptable, because the removal of auto-loop transfer function has rendered these valves to the safety related function of maintaining a flow path integrity only.

Additionally, the removal of the auto-loop transfer and addition of the auto-closure results in the only safety-related function of diesel generator "E" loop A ESW valves' requirement to automatically close during loss of coolant accident (LOCA) and/or Loss of offsite power (LOOP) conditions when diesel

generator is not aligned but is being tested. Therefore, the auto-closure and auto-bypass of the thermal overload protection must be added to Loop A ESW valves for the diesel generator "E". The battery load profile of 125 v dc battery is reduced due to the removal of the auto-loop transfer logic eliminating the load associated with the final minute of the load profile. The staff finds that the licensee's request is acceptable because addition of the requirement of diesel generator "E" loop A ESW valves to automatically close during a LOCA would require these valves to be added to thermal overload protection Table 3.8.4.2.2-1 for the motor operated valves. Also the reduction in the load profile of 125 v dc battery is acceptable because it will continue to be bounded by the previously approved bounding load profile.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that these amendments involve no significant hazards consideration which was published in the Federal Register (54 FR 21314) on May 17, 1989 and consulted with the State of Pennsylvania. No public comments were received, and the State of Pennsylvania did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security nor to the health and safety of the public.

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Dated: August 16, 1989