Docket No. 50-387

Mr. Harold W. Keiser Vice President Nuclear Operations Pennsylvania Power and Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

Dear Mr. Keiser:

SUBJECT: CONFIRMATION OF CHANGE TO TECHNICAL SPECIFICATIONS

RE: SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

This confirms our telephone authorization given on September 14, 1987, for the change to the Technical Specifications for Susquehanna Steam Electric Station, Unit 1, as requested in your letter dated September 14, 1987. Facility Operating License NPF-14 is amended as requested in your letter dated September 14, 1987. The Technical Specification change permits one time relief from the provisions of Technical Specification Section 3.0.4 for the Intermediate Range Monitors. The Corrected Technical Specification page 3/4 3-51 and overleaf page 3/4 3-52 are enclosed.

The formal license amendment, our completed safety evaluation, and the <u>Federal Register</u> Notice of this change to the Technical Specifications for Susquehanna Steam Electric Station, Unit 1, are being processed and copies of these documents will be sent to you in the near future.

Sincerely,

/s/

8709230407 870915 PDR ADDCK 05000387 PDR

Bruce A. Boger, Assistant Director for Region I Reactors Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Enclosures: Technical Specification Pages

cc w/enclosures:
See next page

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# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

September 15. 1987

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Bruce A. Boger, Assistant Director for Region I Reactors

Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Enclosures: Technical Specification Pages

cc w/enclosures: See next page Mr. Harold W. Keiser Pennsylvania Power & Light Company

cc: Jay Silberg, Esq. Shaw, Pittman, Potts & Trowbridge 2300 N Street N.W. Washington, D.C. 20037

Bryan A. Snapp, Esq.
Assistant Corporate Counsel
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. E. A. Heckman Licensing Group Supervisor Pennsylvania Power & Light Company 2 North Ninth Street Allentown, Pennsylvania 18101

Mr. Loren Plisco Resident Inspector P.O. Box 52 Shickshinny, Pennsylvania 18655

Mr. R. J. Benich Services Project Manager General Electric Company 1000 First Avenue King of Prussia, Pennsylvania 19406

Mr. Thomas M. Gerusky, Director Bureau of Radiation Protection Resources Commonwealth of Pennsylvania P. O. Box 2063 Harrisburg, Pennsylvania 17120

Robert W. Alder, Esquire Office of Attorney General P.O. Box 2357 Harrisburg, Pennsylvania 17120

Mr. Jesse C. Tilton, III Allegheny Elec. Coorperative, Inc. 212 Locust Street P.O. Box 1266 Harrisburg, Pennsylvania 17108-1266 Susquehanna Steam Electric Station Units 1 & 2

Mr. W. H. Hirst, Manager
Joint Generation
Projects Department
Atlantic Electric
P.O. Box 1500
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Regional Administrator, Pegion I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

#### INSTRUMENTATION

## 3/4.3.6 CONTROL ROD BLOCK INSTRUMENTATION

## LIMITING CONDITION FOR OPERATION

3.3.6. The control rod block instrumentation channels shown in Table 3.3.6-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.6-2.

APPLICABILITY: As shown in Table 3.3.6-1.

#### ACTION:

- a. With a control rod block instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.6-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.\*
- b. With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, take the ACTION required by Table 3.3.6-1.

## SURVEILLANCE REQUIREMENTS

4.3.6 Each of the above required control rod block trip systems and instrumentation channels shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.6-1.

<sup>\*</sup>For the Intermediate Range Monitors the provisions of Specification 3.0.4 are not applicable for the purposes of entering Operational Condition 5 from Operational Condition 4 on September 14, 1987.

TABLE 3.3.6-1
CONTROL ROD BLOCK INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP FUNCTION	APPLICABLE OPERATIONAL CONDITIONS	ACTION
1. ROD BLOCK MONITOR (a)			
a. Upscale	2	1*	60
<ul><li>b. Inoperative</li><li>c. Downscale</li></ul>	2 2	ן* אן	60 60
2. APRM	2		60
a. Flow Biased Neutron Flu	ıx ~		
Upscale	4	1	61
<pre>b. Inoperative</pre>	4	1, 2, 5	61
c. Downscale	4	1	61
<ul> <li>d. Neutron Flux - Upscale,</li> </ul>	, Startup 4	2, 5	61
3. SOURCE RANGE MONITORS			
<ul> <li>a. Detector not full in (b)</li> <li>b. Upscale (c)</li> </ul>	3	2	61
	3 2 3 2 3 2 3 2	5	61
	3	2	61
	2	5	61
c. Inoperative <sup>(c)</sup>	3	2	61
	2	5	61
d. Downscale <sup>(d)</sup>	3	2 5 2 5 2 5 2	61
	2***	5	61
4. <u>INTERMEDIATE RANGE MONITORS</u>			
<ul> <li>a. Detector not full in</li> </ul>	6	2, 5 2, 5 2, 5 2, 5	61
b. Upscale	6	2, 5	61
c. Inoperative d. Downscale (e)	6	2, 5	61
d. Downscale <sup>(e)</sup>	6	2, 5	61
5. SCRAM DISCHARGE VOLUME			
a. Water Level-High	2	1, 2, 5**	62
6. REACTOR COOLANT SYSTEM RECI	IRCULATION FLOW		
a. Upscale	2	1	62
b. Inoperative	2 2	1	62
c. Comparator	2	1	62