

## 10 CFR 50.46 (a)(3)(i) and (ii)

December 19, 2000  
2130-00-20321

U. S. Nuclear Regulatory Commission  
Attn.: Document Control Desk  
Washington, DC 20555

Subject: Oyster Creek Generating Station (OCGS)  
Docket No. 50-219  
Facility License No. DPR-16  
10 CFR 50.46 Reporting Requirements

Reference: Letter No. 1940-00-20120, "Change in Peak Clad Temperature," from Sander Levin to USNRC dated May 12, 2000

Dear Sir/Madam:

In accordance with 10 CFR 50.46 (a)(3)(i) and (ii), the following is a revision to the licensing basis Loss-of-Coolant Accident (LOCA) peak clad temperature (PCT) for Oyster Creek Generating Station. Table 1 (attached) provides the revised licensing basis PCT value and the PCT error accumulation for Oyster Creek. Due to the sum of the absolute values of the accumulated changes resulting in a value of greater than 50°F, this report is being submitted within 30 days.

On November 20, 2000, AmerGen Energy Co., LLC (AmerGen) received a 10 CFR 50.46 notification letter from GE Nuclear Energy ("Impact of SAFER Time Step Size on the Peak Clad Temperature (PCT) and Local Oxidation for Oyster Creek"), which informed us of a PCT impact due to a change in the size of the time steps used in the SAFER/CORECOOL LOCA analysis methodology. The use of the revised time step criteria results in an increase in the licensing basis PCT of 5°F for the GE9 fuel type at the limiting fuel exposure (an increase of up to 25°F was observed for non-limiting fuel exposures).

Accol

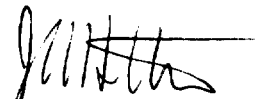
The reference letter had communicated that a 100 gpm leakage flow is being assumed in order to account for slip joint and vent hole leakage in core spray piping. Concurrent with the revised time step criteria, this assumed leakage is being reduced from 100 gpm to 70 gpm. This change in the assumed leakage flow rate results in an additional change to the licensing basis PCT. The reduction in assumed leakage flow results in a 7°F decrease in the licensing basis PCT for the GE9 fuel at the limiting fuel exposure. The revised leakage rate of 70 gpm still bounds the expected leakage rate.

Table 1 provides the baseline PCT, the PCT error accumulations, the PCT analysis input change, and the resultant estimated licensing basis PCT for the GE9 fuel type. The estimated licensing basis PCT is 2173°F, calculated using the conservative estimates for the effect of each of the errors and changes. It should be noted that Global Nuclear Fuel has also performed a limited scope analysis, which computed the licensing basis PCT using the current corrected versions of the models. At the limiting fuel exposure, the GE9 licensing basis PCT was computed to be 2161 °F.

The net effect of the time step change and the reduction in the assumed leakage flow is to decrease the PCT. The current MAPLHGR limits remain bounding, therefore no additional action is required.

If you have any questions, please contact Mr. George B. Rombold at 610-765-5516.

Very truly yours,



J. A. Hutton  
Director - Licensing

Attachments

cc: H. J. Miller, Administrator, USNRC Region I  
L. A. Dudes, USNRC Senior Resident Inspector, Oyster Creek  
H. N. Pastis, USNRC Senior Project Manager, Oyster Creek  
File No. 00090

**TABLE 1**

**CURRENT BASELINE PCT (°F) VALUES AND ERROR ACCUMULATION  
OYSTER CREEK GENERATING STATION**

	GE 9 Fuel
BASELINE PCT	2190
Bottom Head Drain Line Error	+10
CORECOOL Error in Large Central Water Rod Diameter	-55
Core Spray Slip Joint/Vent Hole Leakage Error (100 gpm)	+30
Time Step Size Change	+5
Reduction of Assumed Core Spray Leakage from 100 gpm to 70 gpm	-7
TOTAL	-17
NEW ESTIMATED LICENSING BASIS PCT VALUES	2173