Docket No. 50-387

OCT 24 1984

Mr. Norman W. Curtis
Vice President
Engineering and Construction - Nuclear
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Curtis:

SUBJECT: Amendment No. 25 to Facility Operating License No. NPF-14 - Susquehanna Steam Electric Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No.25 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station, Unit 1. The amendment is in response to your letter dated October 19, 1984. This amendment changes Technical Specification 4.1.3.1.4 regarding scram discharge volume operability by providing a clarification of intent to permit retest of the scram discharge volume vent and drain valves.

A copy of the related safety evaluation supporting Amendment No. 25 to Facility Operating License NPF-14 is enclosed.

Sincerely,

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 25 to NPF-14
2. Safety Evaluation

cc w/enclosures:
See next page

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SUSQUEHANNA

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Susquehanna

cc: Governor's Office of State Planning & Development
Attn: Coordinator, State Clearinghouse
P O. Box 1323
Harrisburg, Pennsylvania 17120

Mr. Bruce Thomas, President
Board of Supervisors
R. D. #1
Berwick, Pennsylvania 18603

U. S. Environmental Protection Agency
Attn: EIS Coordinator
Region III Office
Curtis Building
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

Issuance of Amendment No. 25 to Facility Operating Licence No NPF-14
Susquehanna Steam Electric Station, Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-387

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The applications for amendments filed by the Pennsylvania Power & Light Company, dated October 19, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 25, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of October 19, 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: OCT 24 1984

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| RPerch:dh | EHylton | ASchwencer | JGoldberg | TNovak |
| 10/22/84 | 10/17/84 | 10/22/84 | 10/22/84 | 10/17/84 |

ATTACHMENT TO LICENSE AMENDMENT NO. 25
FACILITY OPERATING LICENSE NO. NPF-14
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

| REMOVE | INSERT |
|---|---|
| 3/4 1-5 | 3/4 1-5 |
| 3/4 1-6 | 3/4 1-6 |

SURVEILLANCE REQUIREMENTS (Continued)

4.1.3.1.4 The scram discharge volume shall be determined OPERABLE by demonstrating:

a. The scram discharge volume drain and vent valves OPERABLE, when control rods are scram tested from a normal control rod configuration of less than or equal to 50% ROD DENSITY at least once per 18 months*, by verifying that the drain and vent valves:

   1. Close within 30 seconds after receipt of a signal for control rods to scram, and

   2. Open when the scram signal is reset.

b. Proper float response by performance of a CHANNEL FUNCTIONAL TEST of the scram discharge volume scram and control rod block level instrumentation after each scram from a pressurized condition.

*The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 2 provided the surveillance is performed within 12 hours after achieving less than or equal to 50% ROD DENSITY.

## CONTROL ROD MAXIMUM SCRAM INSERTION TIMES

### LIMITING CONDITION FOR OPERATION

3.1.3.2 The maximum scram insertion time of each control rod from the fully withdrawn position to notch position 5, based on de-energization of the scram pilot valve solenoids as time zero, shall not exceed 7.0 seconds.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

With the maximum scram insertion time of one or more control rods exceeding 7.0 seconds:

a. Declare the control rod(s) with the slow insertion time inoperable, and

b. Perform the Surveillance Requirements of Specification 4.1.3.2.c at least once per 60 days when operation is continued with three or more control rods with maximum scram insertion times in excess of 7.0 seconds.

Otherwise, be in at least HOT SHUTDOWN within 12 hours.

### SURVEILLANCE REQUIREMENTS

4.1.3.2 The maximum scram insertion time of the control rods shall be demonstrated through measurement with reactor coolant pressure greater than or equal to 950 psig and, during single control rod scram time tests, the control rod drive pumps isolated from the accumulators:

a. For all control rods prior to THERMAL POWER exceeding 40% of RATED THERMAL POWER following CORE ALTERATIONS* or after a reactor shutdown that is greater than 120 days,

b. For specifically affected individual control rods following maintenance on or modification to the control rod or control rod drive system which could affect the scram insertion time of those specific control rods, and

c. For at least 10% of the control rods, on a rotating basis, at least once per 120 days of POWER OPERATION.

*Except movement of SRM, IRM or special movable detectors or normal control rod movement.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

# SAFETY EVALUATION

## AMENDMENT NO. 25 TO NPF-14

## SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1

## DOCKET NO. 50-387

### Introduction

The licensee in a letter dated October 19, 1984, proposed changes to Technical Specification 4.1.3.1.4 of the operating license for Susquehanna Steam Electric Station, Unit 1, regarding scram discharge volume operability, by providing a footnote which indicates that provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 2 provided the surveillance is performed within 12 hours after achieving less than or equal to 50% ROD DENSITY.

### Evaluation

Technical Specification 4.1.3.1.4 presently requires that the scram discharge volume vent and drain valves be demonstrated operable at least once per 18 months by performing a surveillance test which verifies that the scram discharge volume vent and drain valves close within 30 seconds after receipt of a signal for control rods to scram when tested from a normal rod configuration of less than or equal to 50% Rod Density. To perform the test from a normal rod configuration of less than or equal to 50% Rod Density, the unit must be placed in OPERATIONAL CONDITION 2. The present Technical Specification prohibits entering into OPERATIONAL CONDITION 2 without having performed all the surveillance requirements associated with this Technical Specification within the applicable surveillance interval or as otherwise specified. As the Technical Specification is presently written, the unit would have to remain shutdown and could not enter the operational condition necessary to perform the required surveillance to declare the scram discharge volume operable. Therefore, Technical Specification 4.1.3.1.4 requires a clarification in the form of an exemption to Specification 4.0.4 to permit the unit to enter the operation condition necessary to perform the required surveillance to declare the scram discharge volume operable and then be capable of restarting the unit.

This change was issued in the Susquehanna Unit 2 Technical Specifications and other recently licensed BWRs. This change does not constitute a reduction in safety, does not change the frequency or scope of the surveillance performed. The change provides a clarification of intent to provide the flexibility to enter the operational condition necessary to perform the required surveillance tests necessary to demonstrate the system operable.

The NRC staff has made a final determination that the change involves no significant hazards consideration. The change only provides a clarification of intent to retest of the scram discharge volume vent and drain valves by

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providing the flexibility needed to enter the operational condition necessary to perform the surveillance and demonstrate the system operable. This change clearly does not 1) involve a significant increase in the probability or consequences of an accident previously evaluated, 2) create the possibility of a new or different kind of accident from any accident previously evaluated or 3) involve a significant reduction in a margin of safety.

The licensee meets the test of prompt notification. The unanticipated failure of scram discharge volume vent and drain valves under surveillance test on October 18, 1984, was unavoidable and put the facility in a situation requiring the proposed change. The request for change was transmitted by letter (sent via telecopy facilities) to the NRC staff on October 19, 1984.

Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amount of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure because the amendment involves only a clarification of when the surveillance is performed and is not a change in the frequency or scope of the surveillance. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security of the health and safety of the public.

Dated: October 19, 1984