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SEP 3 1982

Docket No.: 50-387

Mr. Norman W. Curtis
Vice President
Engineering and Construction - Nuclear
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Curtis:

Subject: Amendment No. 1 to Facility Operating License No. NPF-14 - Susquehanna Steam Electric Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 1 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station, Unit 1. The amendment is in response to your letters dated August 18, 1982; August 23, 1982; September 1, 1982; and September 2, 1982. This amendment deletes observation well Number 2 from ultimate heat sink surveillance requirements, corrects a typographical error in Technical Specification 4.7.3.c.4, corrects values in Technical Specification 4.6.5.1 provided for surveillance purposes and also provides clarification to the volumes associated with the surveillance testing.

A copy of the related safety evaluation supporting Amendment No. 1 to Facility Operating License NPF-14 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original Signed By:
for Lester L. Kintner

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

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Enclosures:
1. Amendment No. 1 to NPF-14
2. Safety Evaluation
3. Federal Register Notice

cc w/encls.: See next page

No legal objection

OFFICE	DL:LB#2	DL:LB#1	OELD	DL:LB#1			
SURNAME	RPerch/lg	MRushbrook	MCutchin	BJYoungblood			
DATE	9/3/82	9/3/82	9/3/82	9/3/82			

Susquehanna

Mr. Norman W. Curtis
Vice President
Engineering and Construction
Pennsylvania Power & Light Company
Allentown, Pennsylvania 18101

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Capitol Annex
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Department of Environmental Resources
Office of Environmental Planning
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Harrisburg, Pennsylvania 17120

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Post Office Box 2063
Harrisburg, Pennsylvania 17105

Mr. M. A. Dupree
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Post Office Box 5444
Denver, Colorado 80217

EIS Review Coordinator (TECH SPECS ONLY)
EPA Region III
Curtis Building, 6th Floor
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for an amendment filed by the Pennsylvania Power & Light Company, dated August 18, 1982; August 23, 1982; September 1, 1982; and September 2, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
Lester L. Kintner
for B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: SEP 3 1982

No legal objection to form of amendment.

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SURNAME	RPerch/lg	MRushbrook	MCutchin	BJYoungblood			
DATE	9/3/82	9/ 0 /82	9/ 3/82	9/ 3 /82			

ATTACHMENT TO LICENSE AMENDMENT NO. 1
FACILITY OPERATING LICENSE NO. NPF-14
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
3/4 6-31	3/4 6-31
3/4 6-32	3/4 6-32
3/4 7-3	3/4 7-3
3/4 7-4	3/4 7-4
3/4 7-7	3/4 7-7
3/4 7-8	3/4-7-8

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3/4.6.5 SECONDARY CONTAINMENT

SECONDARY CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.5.1 SECONDARY CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3 and *.

ACTION:

Without SECONDARY CONTAINMENT INTEGRITY:

a. In OPERATIONAL CONDITION 1, 2 or 3, restore SECONDARY CONTAINMENT INTEGRITY within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In Operational Condition *, suspend handling of irradiated fuel in the secondary containment, CORE ALTERATIONS and operations with a potential for draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.6.5.1 SECONDARY CONTAINMENT INTEGRITY shall be demonstrated by:

a. Verifying at least once per 24 hours that the pressure within the secondary containment is less than or equal to 0.25 inches of vacuum water gauge.

b. Verifying at least once per 31 days that:

   1. All secondary containment railroad access hatches are closed and sealed or the railroad bay access door is closed.

   2. At least one door in each access to the secondary containment is closed.

   3. All secondary containment penetrations not capable of being closed by OPERABLE secondary containment automatic isolation dampers and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic dampers secured in position.

c. At least once per 18 months:

   1. Verifying that one standby gas treatment subsystem will draw down the secondary containment to greater than or equal to 0.25 inches of vacuum water gauge in less than or equal to 60 seconds, and

   2. Operating one standby gas treatment subsystem for one hour and maintaining greater than or equal to 0.25 inches of vacuum water gauge in the secondary containment at a flow rate of less than or equal to 2885 cfm from Zone I and Zone III.**

*When irradiated fuel is being handled in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

**2000 cfm while the secondary containment interim barrier is installed in Zone III.

SECONDARY CONTAINMENT AUTOMATIC ISOLATION DAMPERS

LIMITING CONDITION FOR OPERATION

3.6.5.2 The secondary containment ventilation system automatic isolation dampers shown in Table 3.6.5.2-1 shall be OPERABLE with isolation times less than or equal to the times shown in Table 3.6.5.2-1.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3 and *.

ACTION:

With one or more of the secondary containment ventilation system automatic isolation dampers shown in Table 3.6.5.2-1 inoperable, maintain at least one isolation damper OPERABLE in each affected penetration that is open and within 8 hours either:

a. Restore the inoperable damper to OPERABLE status, or

b. Isolate each affected penetration by use of at least one deactivated damper secured in the isolation position, or

c. Isolate each affected penetration by use of at least one closed manual valve or blind flange.

Otherwise, in OPERATIONAL CONDITION 1, 2 or 3, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

Otherwise, in Operational Condition *, suspend handling of irradiated fuel in the secondary containment, CORE ALTERATIONS and operations with a potential for draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.6.5.2 Each secondary containment ventilation system automatic isolation damper shown in Table 3.6.5.2-1 shall be demonstrated OPERABLE:

a. Prior to returning the damper to service after maintenance, repair or replacement work is performed on the damper or its associated actuator, control or power circuit by cycling the damper through at least one complete cycle of full travel and verifying the specified isolation time.

b. During COLD SHUTDOWN or REFUELING at least once per 18 months by verifying that on a containment isolation test signal each isolation damper actuates to its isolation position.

c. At least once per 92 days by verifying the isolation time to be within its limit.

*For Zone III dampers when irradiated fuel is being handled in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

ULTIMATE HEAT SINK

LIMITING CONDITION FOR OPERATION

3.7.1.3 The spray pond shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4, 5 and *.

ACTION:

a. With the groundwater level at any spray pond area observation well greater than or equal to 663' MSL, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 10 days outlining the cause of the high groundwater level and the plans for restoring the level to within the limit.

b. With the spray pond otherwise inoperable:

   1. In OPERATIONAL CONDITION 1, 2 or 3, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

   2. In OPERATIONAL CONDITION 4 or 5, declare the RHRSW system and the emergency service water system inoperable and take the ACTION required by Specifications 3.7.1.1 and 3.7.1.2.

   3. In Operational Condition *, declare the emergency service water system inoperable and take the ACTION required by Specification 3.7.1.2. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.3 The spray pond shall be determined OPERABLE by verifying:

a. The average water temperature, which shall be the arithmetical average of the spray pond water temperature at the surface, mid and bottom levels, to be less than or equal to 88°F at least once per 24 hours.

b. The water level at the overflow weir is greater than or equal to 677' mean Sea Level USGS (MSL), at least once per:

   1. 12 hours when water level is $\leq$ 677'6" MSL, and

   2. 14 days when water level is > 677'6" MSL.

c. The groundwater level at observation wells 1, 3, 4, 5, 6, and 1113 to be less than 663' MSL at least once per 31 days.

*When handling irradiated fuel in the secondary containment.

3/4.7.2 CONTROL ROOM EMERGENCY OUTSIDE AIR SUPPLY SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.2 Two independent control room emergency outside air supply system subsystems shall be OPERABLE with each subsystem consisting of:

a. One makeup fan, and

b. One filter train.

APPLICABILITY: All OPERATIONAL CONDITIONS and *.

ACTION:

a. In OPERATIONAL CONDITION 1, 2 or 3 with one control room emergency outside air supply subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In OPERATIONAL CONDITION 4, 5 or *:

   1. With one control room emergency outside air supply subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or initiate and maintain operation of the OPERABLE subsystem in the pressurization mode of operation.

   2. With both control room emergency outside air supply subsystems inoperable, suspend CORE ALTERATIONS, handling of irradiated fuel in the secondary containment and operations with a potential for draining the reactor vessel.

c. The provisions of Specification 3.0.3 are not applicable in Operational Condition *.

SURVEILLANCE REQUIREMENTS

4.7.2 Each control room emergency outside air supply subsystem shall be demonstrated OPERABLE:

a. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the subsystem operates for at least 10 hours with the heaters OPERABLE.

b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the subsystem by:

   1. Verifying that with the subsystem operating at a flow rate of 5810 cfm ± 10% and exhausting through the HEPA filters and charcoal adsorbers, the total bypass flow of the system to the facility vent, including leakage through the subsystem diverting valve, is less than or equal to 1% when the subsystem is tested by admitting cold DOP at the system intake.

*When irradiated fuel is being handled in the secondary containment.

3/4.7.3 REACTOR CORE ISOLATION COOLING SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.3 The reactor core isolation cooling (RCIC) system shall be OPERABLE with an OPERABLE flow path capable of taking suction from the suppression pool and transferring the water to the reactor pressure vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3 with reactor steam dome pressure greater than 150 psig.

ACTION:

With the RCIC system inoperable, operation may continue provided the HPCI system is OPERABLE; restore the RCIC system to OPERABLE status within 14 days or be in at least HOT SHUTDOWN within the next 12 hours and reduce reactor steam dome pressure to less than or equal to 150 psig within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.7.3 The RCIC system shall be demonstrated OPERABLE:

a. At least once per 31 days by:

1. Verifying that the system piping from the pump discharge valve to the system isolation valve is filled with water by:

a. Venting at the high point vents.

b. Performance a CHANNEL FUNCTIONAL TEST of the condensate transfer pump discharge low pressure alarm instrumentation.

2. Verifying that each valve, manual, power operated or automatic in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.

3. Verifying that the pump flow controller is in the correct position.

b. At least once per 92 days by verifying that the RCIC pump develops a flow of greater than or equal to 600 gpm in the test flow path with a system head corresponding to reactor vessel operating pressure when steam is being supplied to the turbine at 920 + 140, - 0 psig.*

*The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.

SURVEILLANCE REQUIREMENTS (Continued)

c. At least once per 18 months by:

1. Performing a system functional test which includes simulated automatic actuation and restart and verifying that each automatic valve in the flow path actuates to its correct position, but may exclude actual injection of coolant into the reactor vessel.

2. Verifying that the system will develop a flow of greater than or equal to 600 gpm in the test flow path when steam is supplied to the turbine at a pressure of 150, + 15, -0 psig.*

3. Verifying that the suction for the RCIC system is automatically transferred from the condensate storage tank to the suppression pool on a condensate storage tank water level-low signal.

4. Performing a CHANNEL CALIBRATION of the condensate transfer pump discharge low pressure alarm instrumentation and verifying the low pressure alarm setpoint to greater than or equal to 113 psig.

*The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the tests.

SAFETY EVALUATION
AMENDMENT NO. 1 TO NPF-14
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-387

Introduction

The licensee proposed changes to the Technical Specifications of the operating license for Susquehanna Steam Electric Station, Unit 1 which are as follows:

a) In Specification 4.7.1.3.c, delete observation well Number 2 from ultimate heat sink surveillance requirements

b) In Specification 4.7.3.c, correct a typographical error on the setpoint for the low pressure alarm instrumentation on the condensate transfer pump discharge

c) In Specification 4.6.5.1.c.2, change the test flow rates to bring the values back within the assumptions used in the LOCA offsite dose analysis, and clarify the volumes associated with the secondary containment surveillance testing.

Evaluation

a) Observation Well Number 2

In Specification 4.7.1.3.c, the licensee requested a change in the groundwater measurements from "at each of the seven observation wells" to read "at observation wells 1, 3, 4, 5, 6, and 1113". The licensee has also requested deletion of footnote**. In support of this request the licensee provided in a letter dated August 3, 1982, an analysis on the high water level in piezometer Number 2. In this analysis, the licensee indicated the southwest portion of the spraypond is cut into bedrock and the remainder of the spraypond is supported on soil. The piezometers are monitored to assure that remedial actions are taken if groundwater elevations around the spraypond rise to levels high enough to present a liquefaction potential in the event of an earthquake. Liquefaction relates only to granular soil. The six wells identified in the proposed technical specification are located in soil and are thus relevant to the purposes of the technical specification. Since piezometer Number 2 is located in an area where the spraypond is supported on bedrock, the licensee concluded that piezometer Number 2 cannot be used to assess the possibility of liquefaction occurring. The staff has evaluated and concurs with the licensee's analysis. High groundwater levels have been observed at this well. However, high groundwater levels at this location are irrelevant to the purpose of the technical specification. Therefore, deletion of piezometer Number 2 from surveillance requirements for groundwater measurement is acceptable.

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b) Condensate Transfer Pump Discharge Low Pressure Alarm

In Specification 4.7.3.c.4, the licensee requested a change to the low pressure alarm setpoint from "less than or equal 113 psig" to read greater than or equal to 113 psig". The current specification setpoint is a typographical error and is inappropriate for a decreasing pressure alarm. The setpoint for the low pressure alarm on the condensate transfer pump discharge is correctly stated in Specification 4.5.1.c.5. Therefore, the staff finds this change to be acceptable.

c) Standby Gas Treatment System Test Flow Rates

In Specification 4.6.5.1.c.2, the licensee requested a change to the test flow rate from "3050 cfm ± 10%" to read "less than or equal to 2885 cfm", and a change to the associated footnote from "2300 cfm" to "2000 cfm". The offsite dose analysis assumes a 100% air change per day in the secondary containment. The licensee has identified the 3050 cfm and the 2300 cfm values to be based on preliminary rather than final calculations. The proposed flow rates of 2885 cfm and 2000 cfm correspond to the free air volumes of Unit 1 secondary containment (Zone I and III) and Unit 1 interim secondary containment (Zone I and with the interim barrier installed in Zone III), respectively, for a 100% air change per day. Since the bases for the requested changes are consistent with the rationale and justification used in the formulation of the original technical specification, the staff finds the change to the test flow rates to be acceptable.

In Specification 4.6.5.1.c.2, the licensee also requested a change from "for both Units 1 and 2" to read "from Zone I and Zone III", and a change to the associated footnote from "while Unit 2 secondary containment is isolated from Unit 1 secondary containment" to read "while the secondary containment interim barrier is installed in Zone III". The proposed changes clarify the intent of secondary containment testing of Unit 1 with and without the interim barrier in Zone III. Testing of the Unit 2 secondary containment will be covered by Unit 2 Technical Specifications. The staff has reviewed the licensee's justification and analyses and finds the changes clarifying the secondary containment for Unit 1 acceptable.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this statement.

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Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 3, 1982

*SEE PREVIOUS ORC FOR CONCURRENCES.

OFFICE	DL:LB#2	DL:LB#1*	DE:HGEB*	DL:LB#1			
SURNAME	RPerch/lg	MRushbrook	MFliegel	BJYoungblood			
DATE	9/3/82	9/ /82	9/ /82	9/3/82			

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 3, 1982

[illegible] not create the possibility of an accident of a type different from any evaluated previously.

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SURNAME	RPerch/lg	MRushbrook	MFliegel	BJYoungblood			
DATE	9/3/82	9/3/82	9/3/82	9/ /82			

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-387

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

NOTICE OF ISSUANCE OF AMENDMENT OF FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. NPF-14, issued to Pennsylvania Power & Light Company and Allegheny Electric Cooperative, Inc., for Susquehanna Steam Electric Station, Unit 1 (the facility) located in Luzerne County, Pennsylvania. This amendment grants changes to Technical Specifications to delete observation well Number 2 from ultimate heat sink surveillance requirements, to correct a typographical error in the low pressure alarm setpoint for the condensate transfer pump, and to correct values on standby gas treatment flow rates provided for surveillance purposes and to clarify the volumes associated with the same surveillance test. This amendment is effective as of the date of issuance.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For futher details with respect to this action, see (1) the application for the amendment dated August 18, 1982; August 23, 1982; September 1, 1982; and September 2, 1982; (2) Amendment No. 1 to License NPF-14 dated September 3, 1982; and (3) the Commission's evaluation dated September 3, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, and at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, Pennsylvania 18701. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 3rd day of September 1982.

FOR THE NUCLEAR REGULATORY COMMISSION
Original Signed By:
Lester L. Kintner

L. L. Kintner, Acting Branch Chief
Licensing Branch No. 1
Division of Licensing

No legal objection to form of notice

LLK

OFFICE	DL:LB#2 RP	DL:LB#1	OELD	DL:LB#1			
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