



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

July 9, 1992

Docket No. 50-286

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M83195)

The Commission has issued the enclosed Amendment No. 123 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 10, 1992.

The amendment revises Environmental Technical Specifications, Part II, Section 2.7 (Radiological Environmental Monitoring Program) and Section 3.7 (Radiological Environmental Monitoring Program Surveillance Requirements). These sections have been revised to specify lower limits of detection (LLD) and reporting requirements for iodine-131 in environmental samples of non-drinking water. In addition, the amendment corrects an administrative error in Section 3.7 and Tables 2.7-2 and 3.7-1 have been reformatted for consistency.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "N. F. Conicella".

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.123 to DPR-64
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. Ralph E. Beedle
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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DATED: July 9, 1992

AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT UNIT 3

Docket File

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cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 10, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

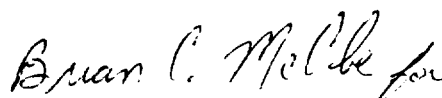
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 123, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 123

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix B, Part II as follows:

Remove Pages

2.7-9

3.7-2

3.7-3

Insert Pages

2.7-9

3.7-2

3.7-3

TABLE 2.7-2

REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES					
REPORTING LEVELS					
Analysis	Water (pCi/l)	Airborne Particulate or Gases (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/l)	Food Products (pCi/kg, wet)
H-3	20,000*				
Mn-54	1,000		30,000		
Fe-59	400		10,000		
Co-58	1,000		30,000		
Co-60	300		10,000		
Zn-65	300		20,000		
Zr-Nb-95	400				
I-131	2**	0.9		3	100
Cs-134	30	10	1,000	60	1,000
Cs-137	50	20	2,000	70	2,000
Ba-La-140	200			300	

* For drinking water samples. This is 40 CFR Part 141 value. If no drinking water pathway exists, a value of 30,000 pCi/l may be used.

** If no drinking water pathway exists, a value of 20 pCi/l may be used.

TABLE 3.7-1

DETECTION CAPABILITIES FOR ENVIRONMENTAL SAMPLE ANALYSIS ^a						
LOWER LIMIT OF DETECTION (LLD) ^{b,c}						
Analysis	Water (pCi/ℓ)	Airborne Particulate or Gases (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/ℓ)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
gross beta	4	0.01				
H-3	2,000*					
Mn-54	15		130			
Fe-59	30		260			
Co-58, 60	15		130			
Zn-65	30		260			
Zr-Nb-95	15					
I-131	1**	0.07		1	60	
Cs-134	15	0.05	130	15	60	150
Cs-137	18	0.06	150	18	80	180
Ba-La-140	15			15		

* If no drinking water pathway exists, a value of 3,000 pCi/ℓ may be used.

** If no drinking water pathway exists, a value of 15 pCi/ℓ may be used.

TABLE 3.7-1 (Continued)

TABLE NOTATION

- a This list does not mean that only these nuclides are to be considered. Other peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radiological Environmental Operating Report pursuant to Specification 5.3.2.2.
- b Required detection capabilities for thermoluminescent dosimeters used for environmental measurements are given in Regulatory Guide 4.13.
- c The LLD is defined, for purposes of these specifications as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal. Equations used in the calculation of the LLD for a particular measurement system are presented in the ODCM.

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement. Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally background fluctuations, unavoidable small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Operating Report pursuant to Specification 5.3.2.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated April 10, 1992, the Power Authority of the State of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3), Environmental Technical Specifications (ETS). The ETS are contained in Appendix B to the facility operating license. Part I of the ETS contains the non-radiological environmental protection plan and Part II of the ETS contains the radiological environmental requirements. The requested changes would revise ETS, Part II, Section 2.7 (Radiological Environmental Monitoring Program) and Section 3.7 (Radiological Environmental Monitoring Program Surveillance Requirements). These sections would be revised to specify lower limits of detection (LLD) and reporting requirements for iodine-131 (I-131) in environmental samples of non-drinking water. In addition, the amendment would correct an administrative error in Section 3.7 and Tables 2.7-2 and 3.7-1 would be reformatted for consistency.

Generic Letter 89-01, "Implementation of Programmatic Control for Radiological Effluent Technical Specifications in the Administrative Controls Section of the Technical Specifications and the Relocation of Procedural Details of RETS [radiological effluent technical specifications] to the Offsite Dose Calculation Manual or to the Process Control Program" was issued on January 31, 1989. By NRC letter dated November 14, 1990, NUREG-1301, "Offsite Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for Pressurized Water Reactors (Generic Letter 89-01, Supplement No. 1)" was forwarded to all PWR licensees. The letter stated that NUREG-1301 superseded prior draft RETS guidance for those licensees who elected to implement Generic Letter 89-01. The prior RETS guidance was contained in draft NUREG-0472, Revision 3, "Radiological Effluent Technical Specifications for Pressurized Water Reactors." The November 14, 1990, letter also stated that those not electing to implement Generic Letter 89-01 should continue to follow the prior draft RETS guidance. The licensee has not implemented Generic Letter 89-01.

2.0 EVALUATION

Table 2.7-2 of ETS, Part II, lists the reporting levels for radioactivity concentrations in environmental samples. The current ETS reporting requirement for I-131 is 2 pCi/liter. The current ETS does not provide a clear reporting requirement for I-131 levels in non-drinking water samples.

To clarify the ETS reporting requirements for I-131 in non-drinking water samples, the licensee proposed changing ETS Table 2.7-2 to indicate a limit of 20 pCi/liter if no drinking water pathway exists and maintaining the 2 pCi/liter limit if a drinking water pathway exists. The licensee states that the proposed reporting requirement for I-131 in non-drinking water samples is consistent with the I-131 reporting requirement provided by the NRC in draft NUREG-0472, Revision 3.

The NRC staff has reviewed the proposed changes to Table 2.7-2 and concludes that the proposed reporting limit for I-131 in non-drinking water samples is consistent with the values provided in NUREG-1301 and NUREG-0472. Therefore, the staff finds the proposed change acceptable.

Table 3.7-1 of ETS, Part II, lists the detection capabilities for environmental sample analysis. The current ETS states if no drinking water pathway exists, the I-131 gamma isotopic analysis lower limit of detection (LLD) may be used. The licensee states that the value of gamma isotopic LLD obtained for I-131 in non-drinking water samples varies from test to test. Therefore, the licensee proposed changing ETS Table 3.7-1 to indicate a specific LLD value of 15 pCi/liter if no drinking water pathway exists and maintaining the current LLD value of 1 pCi/liter if a drinking water pathway exists. The licensee states that the proposed LLD value for I-131 in non-drinking water samples is consistent with the I-131 LLD values provided by the NRC in draft NUREG-0472, Revision 3.

The NRC staff has reviewed the proposed changes to Table 3.7-1 and concludes that the proposed LLD value for I-131 in non-drinking water samples is consistent with the values provided in NUREG-1301 and NUREG-0472. Therefore, the staff finds the proposed change acceptable.

The licensee proposed an administrative change to Notes "a" and "c" of Table 3.7-1 of ETS, Part II. The change would specify Section 5.3.2.2 as the ETS section that discusses the Annual Radiological Environmental Operating Report. The current ETS incorrectly specifies Section 5.3.3.2. In addition, the licensee proposed reformatting Tables 2.7-2 and 3.7-1 for consistency.

The NRC staff has reviewed the proposed administrative changes and finds them acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance and reporting requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 22265). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Nicola F. Conicella

Date: July 9, 1992

July 9, 1992

Docket No. 50-286
Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original Signed By
Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No.123 to DPR-64
- 2. Safety Evaluation

cc w/enclosures:
See next page

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DATE	6/11/92	6/11/92	6/15/92	6/18/92	7/9/92

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