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Docket No.: 50-387

Mr. Norman W. Curtis
Vice President
Engineering and Construction - Nuclear
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Curtis:

Subject: Amendment No. 4 to Facility Operating License No. NPF-14 -
Susquehanna Steam Electric Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 4 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station, Unit 1. The amendment is in response to your letters dated October 6, 1982, October 15, 1982 and October 20, 1982. This amendment grants changes to the Technical Specifications to change the isolation logic for the Containment Instrument Gas, Reactor Building Closed Cooling Water and Reactor Building Chilled Water isolation valves that are affected to isolate on Reactor Pressure Vessel Level 2 or drywell pressure, and also isolate in the event of a loss of offsite power.

A copy of the related safety evaluation supporting Amendment No. 4 to Facility Operating License NPF-14 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 4 to NPF-14
2. Safety Evaluation
3. Federal Register Notice

cc w/encls.: See next page

No legal defect

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PDR ADOCK 05000387
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OFFICE	DL:LB#1	DL:LB#2	OELB	DL:LB#1	DL:AD/L		
SURNAME	MRushbrook:cw	ABournia	CURCHIN	BJYoungblood	TMNovak		
DATE	10/27/82	10/22/82	10/27/82	10/27/82	10/22/82		

Susquehanna

OCT 22 1982

Mr. Norman W. Curtis
Vice President
Engineering and Construction
Pennsylvania Power & Light Company
Allentown, Pennsylvania 18101

ccs: Jay Silberg, Esquire
Shaw, Pittman, Potts & Trowbridge
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General Counsel and Secretary
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Environmental Coalition on Nuclear Power
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505 Executive House
P. O. Box 2357
Harrisburg, Pennsylvania 17120

Mr. R. Haynes, Administration
U. S. Nuclear Regulatory Commission -
Region I
631 Park Avenue
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OCT 22 1982

President, Board of Supervisors
728 East Third Street
Berwick, Pennsylvania

Attorney General
Department of Justice
Capitol Annex
Harrisburg, Pennsylvania 17120

Department of Environmental Resources
Office of Environmental Planning
ATTN: Mr. David Hess
Room 813, Executive House
Harrisburg, Pennsylvania 17120

Department of Environmental Resources
ATTN: Director, Office of Radiological Health
P. O. Box 2063
Harrisburg, Pennsylvania 17105

Mr. M. A. Dupree
Mobil Oil Corporation
P. O. Box 5444
Denver, Colorado 80217

EIS Review Coordinator (TECH SPECS ONLY)
EPA Region III
Curtis Building, 5th Floor
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-14
 Amendment No. 4

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The applications for amendment filed by the Pennsylvania Power & Light Company, dated October 6, 1982 (PLA-1329), October 15, 1982 (PLA-1349) and October 20, 1982 (PLA-1356) comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 4, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

sd
B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: OCT 22 1982

*No legal objection
to issue of amendment
to DER license not
requested.*

OFFICE ▶	DL:LB#1	DL:LB#2	OELD	DL:LB#1			
SURNAME ▶	MRushbrook:cv	ABournia	<i>CUTCHIN</i>	BJYoungblood			
DATE ▶	10/2/82	10/2/82	10/2/82	10/2/82			

ATTACHMENT TO LICENSE AMENDMENT NO. 4
FACILITY OPERATING LICENSE NO. HPF-14
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 3-11
 3/4 3-17

3/4 3-21
 3/4 3-23
 3/4 6-20

INSERT

3/4 3-11
 3/4 3-17

3/4 3-21
 3/4 3-23
 3/4 6-20

OFFICE ▶							
SURNAME ▶							
DATE ▶							

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL(s)^(a)</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level				
1) Low, Level 3	A	2	1, 2, 3	20
2) Low Low, Level 2	B	2	1, 2, 3	20
3) Low Low Low, Level 1	X	2	1, 2, 3	20
b. Drywell Pressure - High	Y,Z,X	2	1, 2, 3	20
c. Manual Initiation	NA	1	1, 2, 3	24
<u>2. SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level - Low Low, Level 2	Y (c)	2	1, 2, 3 and *	25
b. Drywell Pressure - High	Y,Z (c)	2	1, 2, 3	25
c. Refuel Floor High Exhaust Duct Radiation - High	**	2	1, 2, 3 and *	25
d. Railroad Access Shaft Exhaust Duct Radiation - High	**	2	1, 2, 3 and *	25
e. Refuel Floor Wall Exhaust Duct Radiation - High	**	2	1, 2, 3 and *	25
f. Manual Initiation	NA	1	1, 2, 3 and *	24

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level		
1) Low, Level 3	> 13.0 inches*	> 11.5 inches
2) Low Low, Level 2	> -38.0 inches*	> -45.0 inches
3) Low Low Low, Level 1	> -129 inches*	> -136 inches
b. Drywell Pressure - High	< 1.72 psig	< 1.88 psig
c. Manual Initiation	NA	NA
<u>2. SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low, Level 2	≥ -38.0 inches*	≥ -45.0 inches
b. Drywell Pressure - High	≤ 1.72 psig	≤ 1.88 psig
c. Refuel Floor High Exhaust Duct Radiation - High	≤ 2.5 mR/hr.**	≤ 4.0 mR/hr.**
d. Railroad Access Shaft Exhaust Duct Radiation - High	≤ 2.5 mR/hr.**	≤ 4.0 mR/hr.**
e. Refuel Floor Wall Exhaust Duct Radiation - High	≤ 2.5 R/hr.**	≤ 4.0 mR/hr.**
f. Manual Initiation	NA	NA
<u>3. MAIN STEAM LINE ISOLATION</u>		
a. Reactor Vessel Water Level - Low Low, Level 2	≥ -38 inches*	≥ -45.0 inches
b. Main Steam Line Radiation - High	≤ 3 X full power background	≤ 3.6 X full power background
c. Main Steam Line Pressure - Low	≥ 861 psig	≥ 841 psig
d. Main Steam Line Flow - High	≤ 107 psid	≤ 110 psid

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level	
1) Low, Level 3	<10 ^(a)
2) Low Low, Level 2	<1.0*/<10 ^{(a)**}
3) Low Low Low, Level 1	<10 ⁽²⁾
b. Drywell Pressure - High	<10 ^(a)
c. Manual Initiation	NA
2. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level-Low Low, Level 2	<10 ^(a)
b. Drywell Pressure - High	<10 ^(a)
c. Refuel Floor High Exhaust Duct Radiation - High ^(b)	<10 ^(a)
d. Railroad Access Shaft Exhaust Duct Radiation - High ^(b)	<10 ^(a)
e. Refuel Floor Wall Exhaust Duct Radiation -High ^(b)	<10 ^(a)
f. Manual Initiation	NA
3. <u>MAIN STEAM LINE ISOLATION</u>	
a. Reactor Vessel Water Level- Low Low, Level 2	<10 ^(a)
b. Main Steam Line Radiation - High ^{(a)(b)}	<1.0*/<10 ^{(a)**}
c. Main Steam Line Pressure - Low	<1.0*/<10 ^{(a)**}
d. Main Steam Line Flow-High	<0.5*/<10 ^{(a)**}
e. Condenser Vacuum - Low	NA
f. Main Steam Line Tunnel Temperature - High	NA
g. Main Steam Line Tunnel Δ Temperature - High	NA
h. Manual Initiation	NA
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. RWCS Δ Flow - High	<10 ^{(a)##}
b. RWCS Area Temperature - High	NA
c. RWCS Area Ventilation Temperature ΔT - High	NA
d. SLCS Initiation	NA
e. Reactor Vessel Water Level - Low Low, Level 2	<10 ^(a)
f. RWCS Δ Pressure - High	NA
g. Manual Initiation	NA
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>	
a. RCIC Steam Line Δ Pressure - High	<10 ^{(a)###}
b. RCIC Steam Supply Pressure - Low	<10 ^(a)
c. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
d. RCIC Equipment Room Temperature - High	NA
e. RCIC Equipment Room Δ Temperature - High	NA
f. RCIC Pipe Routing Area Temperature - High	NA
g. RCIC Pipe Routing Area Δ Temperature - High	NA
h. RCIC Emergency Area Cooler Temperature - High	NA
i. Manual Initiation	NA

TABLE 4.3.2.1-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level -				
1) Low, Level 3	S	M	R	1, 2, 3
2) Low Low, Level 2	S	M	R	1, 2, 3
3) Low Low Low, Level 1	S	M	R	1, 2, 3
b. Drywell Pressure - High	NA	M	R	1, 2, 3
c. Manual Initiation	NA	R	NA	1, 2, 3
<u>2. SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	1, 2, 3 and *
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. Refuel Floor High Exhaust Duct Radiation - High	S	M	R	1, 2, 3 and *
d. Railroad Access Shaft Exhaust Duct Radiation - High	S	M	R	1, 2, 3 and *
e. Refuel Floor Wall Exhaust Duct Radiation - High	S	M	R	1, 2, 3 and *
f. Manual Initiation	NA	R	NA	1, 2, 3 and *

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>ISOLATION SIGNAL(s)^(a)</u>
<u>Automatic Isolation Valves^(b) (Continued)</u>		
<u>Containment Instrument Gas</u>		
HV-12603	20	X
SV-12605	N/A	X
SV-12651	N/A	X
SV-12661	N/A	Y
SV-12671	N/A	Y
<u>RBCCW</u>		
HV-11313	30	X
HV-11314	30	X
HV-11345	30	X
HV-11346	30	X
<u>Containment Purge</u>		
HV-15703	19	Y,R
HV-15704	19	Y,R
HV-15705	5	Y,R
HV-15711	5	Y,R
HV-15713	30	Y,R
HV-15714	30	Y,R
HV-15721	6	Y,R
HV-15722	30	Y,R
HV-15723	30	Y,R
HV-15724	19	Y,R
HV-15725	19	Y,R
<u>RHR - Drywell Spray^(f)</u>		
HV-151F016 A,B	90	G
<u>RB Chilled Water</u>		
HV-18781 A1,A2,B1,B2	40	X
HV-18782 A1,A2,B1,B2	6	X
HV-18791 A1,A2,B1,B2	15	Y
HV-18792 A1,A2,B1,B2	4	Y

SAFETY EVALUATION
AMENDMENT NO. 4 TO LICENSE NPF-14
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-367

INTRODUCTION

By letters, dated October 6, 1982, October 15, 1982, and October 20, 1982 Pennsylvania Power and Light Company (licensee) has proposed certain changes to the Technical Specifications for the Susquehanna Steam Electric Station Unit 1. The intent of the change is to add information to the Technical Specifications which will minimize the possibility of equipment damage due to high drywell temperature.

EVALUATION

The licensee states that the current Technical Specifications regarding the isolation logic for the Containment Instrument Gas, Reactor Building Closed Cooling Water and Reactor Building Chilled Water requires isolation of the affected valves on receipt of a signal indicating Reactor Pressure Vessel (RPV) Level 2 or high drywell pressure and also in the event of a loss of offsite power (LOOP). Isolating these lines (in non-accident cases) might result in the maximum drywell temperature being higher than existing non-accident thermal qualification limits for equipment. To minimize this possibility the licensee proposed to change the logic to provide isolation on RPV Level 1 and/or high drywell pressure. The licensee proposes that in the event of a LOOP, these valves would be reopened once the diesel generator has started.

In this regard, we note that the licensee's proposal provides a reasonable isolation provision that meets the provisions of Standard Review Plan 5.2.4 and TH-Action Plan II.E.4-2 and will not increase the consequences of an accident and at the same time maintain containment cooling for non-accident isolation events.

We, therefore, conclude that the proposed change should be approved provided that in case of containment isolation following a LOOP, reopening of these valves would be only after the diesel generator has started and that the RPV Level 1 and/or the drywell pressure signals have been cleared. This provision is in the logic.

ENVIRONMENTAL CONSIDERATION

We have determined that this amendment does not authorize a change in effluent types of total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this statement.

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DATE							
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	PDR ADOCK	05000387					
	P	PDR					

CONCLUSION

We have concluded, based on the considerations discussed above, that; (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: OCT 22 1982

OFFICE	DL:LB#2	DL:LB#1					
SURNAME	ABournia:cw	BJYoungblood					
DATE	10/24/82	10/24/82					

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-387

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

NOTICE OF ISSUANCE OF AMENDMENT OF FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 4 to Facility Operating License No. NPF-14, issued to Pennsylvania Power & Light Company and Allegheny Electric Cooperative, Inc., for Susquehanna Steam Electric Station, Unit 1 (the facility) located in Luzerne County, Pennsylvania. This amendment grants changes to Technical Specifications to change the isolation logic for the Containment Instrument Gas, Reactor Building Closed Cooling Water and Reactor Building Chilled Water isolation valves that are affected to isolate on Reactor Pressure Vessel Level 2 or high drywell pressure, and also isolate in the event of a loss of offsite power. This amendment is effective as of the date of issuance.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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DATE ▶

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for the amendment dated October 6, 1982, October 15, 1982, and October 20, 1982; (2) Amendment No. 4 to License NPF-14 dated October 22, 1982; and (3) the Commission's evaluation dated October 22, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, and at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, Pennsylvania 18701. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

*No legal objection to issue of notice
DER review not requested*

OFFICE	DL:LB#1	DL:LB#2	OELD	DL:LB#1			
SURNAME	MRushbrook:cv	ABournia	CUTCHIN	BJYoungblood			
DATE	10/27/82	10/24/82	10/27/82	10/27/82			