



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 20, 2000

Mr. T. F. Plunkett
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS
REGARDING 480-VOLT LOAD CENTERS DEGRADED VOLTAGE
PROTECTION SCHEMES (TAC NOS. MA9824 AND MA9825)

Dear Mr. Plunkett:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 209 to Facility Operating License No. DPR-31 and Amendment No. 203 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Units 3 and 4, respectively. The proposed amendments would revise the current requirements of Technical Specifications (TSs) 3.3-2, Items 7.b and 7.c. The amendments consist of changes to the ACTION Statement 18 to allow operation of the units with both channels of undervoltage protection bypassed for up to 8 hours to allow performance of the monthly surveillance without placing the units in a condition not permitted by the TSs. In addition, the amendments also authorize an administrative change to Item 7.b. of TS Tables 3.3-2, 3.3-3, and 4.3-2 modifying "Degraded Voltage" to "Undervoltage" to make it consistent with the Updated Final Safety Analysis Report description. The amendments are in response to your application dated August 18, 2000.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Kahtan N. Jabbour, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosures:

1. Amendment No. 209 to DPR-31
2. Amendment No. 203 to DPR-41
3. Safety Evaluation

cc w/enclosures: See next page

NRR-058

Mr. T. F. Plunkett
 President - Nuclear Division
 Florida Power and Light Company
 P.O. Box 14000
 Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS
 REGARDING 480 VOLTS LOAD CENTERS DEGRADED VOLTAGE
 PROTECTION SCHEMES (TAC NOS. MA9824 AND MA9825)

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/RA by R. Hernan for/
 Kahtan N. Jabbour, Senior Project Manager, Section 2
 Project Directorate II
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Enclosures:

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cc w/enclosures: See next page

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Mr. T. F. Plunkett
Florida Power and Light Company

TURKEY POINT PLANT

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY
DOCKET NO. 50-250
TURKEY POINT PLANT UNIT NO. 3
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated August 18, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

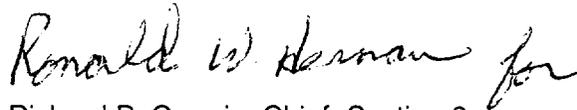
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

B. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 209, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- b. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 20, 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 203
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated August 18, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

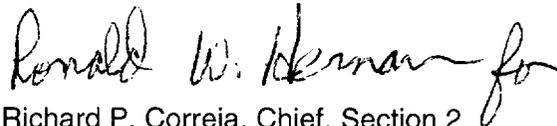
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.203 , are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- c. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 20, 2000

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO.209 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO.203 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove pages

3/4 3-19

3/4 3-22

3/4 3-28

3/4 3-33a

Insert pages

3/4 3-19

3/4 3-22

3/4 3-28

3/43-33a

TABLE 3.3-2 (Continued)
ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>		<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
6.	Auxiliary Feedwater### (Continued)					
b.	Stm. Gen. Water Level-- Low-Low	3/steam generator	2/steam generator in any steam generator	2/steam generator	1, 2, 3	15
c.	Safety Injection	See Item 1. above for all Safety Injection initiating functions and requirements.				
d.	Bus Stripping	1/bus	1/bus	1/bus	1, 2, 3	23
e.	Trip of All Main Feed-water Pumps Breakers	1/breaker	(1/breaker) /operating pump	(1/breaker) /operating pump	1, 2	23
7.	Loss of Power					
a.	4.16 kV Busses A and B (Loss of Voltage)	2/bus	2/bus	2/bus	1, 2, 3, 4	18
b.	480 V Load Centers 3A, 3B, 3C, 3D and 4A, 4B, 4C, 4D Undervoltage	2 per load center	2 on any load center	2 per load center	1, 2, 3, 4	18
	Coincident with: Safety Injection	See Item 1. above for all Safety Injection initiating functions and requirements				

TABLE 3.3-2 (Continued)

TABLE NOTATION (Continued)

- ACTION 18 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the inoperable channel is placed in the tripped condition within 6 hours. Both channels of any one load center may be taken out of service for up to 8 hours in order to perform surveillance testing per Specification 4.3.2.1.
- ACTION 19 - With less than the Minimum Number of Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.
- ACTION 20 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 8 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 21 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the ACTION required by Specification 3.7.1.5.
- ACTION 22 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours; however, one channel may be bypassed for up to 8 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 23 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, comply with Specification 3.0.3.
- ACTION 24 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, within 1 hour isolate the control room Emergency Ventilation System and initiate operation of the Control Room Emergency Ventilation System in the recirculation mode.
- ACTION 25 - With number of OPERABLE channels one less than the Total number of channels, STARTUP and/or POWER OPERATION may proceed provided the inoperable channel is placed in the tripped condition within 6 hours. For subsequent required DIGITAL CHANNEL OPERATIONAL TESTS the inoperable channel may be placed in bypass status for up to 4 hours.

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM
INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>ALLOWABLE VALUE #</u>	<u>TRIP SETPOINT</u>
7. Loss of Power (Continued)		
b. 480V Load Centers Undervoltage		
<u>Load Center</u>		
3A	[]	430V ±5V (10 sec ±1 sec delay)
3B	[]	438V ±5V (10 sec ±1 sec delay)
3C	[]	434V ±5V (10 sec ±1 sec delay)
3D	[]	434V ±5V (10 sec ±1 sec delay)
4A	[]	435V ±5V (10 sec ±1 sec delay)
4B	[]	434V ±5V (10 sec ±1 sec delay)
4C	[]	434V ±5V (10 sec ±1 sec delay)
4D	[]	430V ±5V (10 sec ±1 sec delay)
Coincident with: Safety Injection and	See Item 1. above for all Safety Injection Allowable Values.	See Item 1. above for all Safety Injection Trip Setpoints.
Diesel Generator Breaker Open	N.A.	N.A.

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

<u>CHANNEL FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST#</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
6. Auxiliary Feedwater (Continued)						
c. Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.					
d. Bus Stripping	N.A.	R	N.A.	R	N.A.	1, 2, 3
e. Trip of All Main Feedwater Pump Breakers.	N.A.	N.A.	N.A.	R	N.A.	1, 2
7. Loss of Power						
a. 4.16 kV Busses A and B (Loss of Voltage)	N.A.	R	N.A.	R	N.A.	1, 2, 3, 4
b. 480V Load Centers 3A, 3B, 3C, 3D and 4A, 4B, 4C, 4D Undervoltage	S	R	N.A.	M(1)	N.A.	1, 2, 3, 4
Coincident with: Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.					
c. 480V Load Centers 3A, 3B, 3C, 3D and 4A, 4B, 4C, 4D Degraded Voltage	S	R	N.A.	M(1)	N.A.	1, 2, 3, 4



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 209 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 203 TO FACILITY OPERATING LICENSE NO. DPR-41
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

1. INTRODUCTION

By letter dated August 18, 2000, Florida Power and Light Company (FPL or the licensee) proposed to amend the Facility Operating Licenses and Technical Specifications (TSs) for Turkey Point Plant, Units 3 and 4. In accordance with Title 10, *Code of Federal Regulations* (10 CFR), Section 50.90, FPL requested that Appendix A of Facility Operating Licenses DPR-31 and DPR-41 be amended to modify the Turkey Point Units 3 and 4 TS Section 3.3.2, Engineered Safety Features Actuation System Instrumentation, Table 3.3-2, ACTION Statement 18. This change is required to allow performance of the monthly surveillance without placing the plant in a condition prohibited by the TS, due to having the number of operable channels two less than the total number of channels, a condition prohibited by the TS. In addition, the amendment also requests an administrative change to Item 7.b. of TS Tables 3.3-2, 3.3-3 and 4.3-2 modifying "Degraded Voltage" to "Undervoltage" to make it consistent with the Updated Final Safety Analysis Report (UFSAR) description.

2.0 BACKGROUND

Turkey Point Units 3 and 4 TS Table 3.3-2, Loss of Power, Items 7.b and 7.c, addresses the requirements for the 480-volt load centers degraded voltage protection. Under the existing TS, the minimum required operable channels is "2 per load center" which is equal to the number of channels available. A test switch is used to test the operability of the relays periodically, in accordance with the TSs. During the performance of the monthly surveillance, the test switch is placed in the test position multiple times for no more than 90 seconds each time. During this time, when the switch is in the test position, both channels of the load center being tested are rendered inoperable. Table 3.3-2, Loss of Power, Items 7.b and 7.c and the related ACTION

Enclosure

Statement 18, do not address this condition. As a result, TS 3.0.3 is applicable for the time the monthly surveillance is being performed. FPL submitted Licensee Event Report 250/2000-02 to document the previously unrecognized entries into TS 3.0.3.

3.0 EVALUATION

Each of the proposed changes is stated and discussed below in detail.

The undervoltage monitoring system on the 480-volt safety related load centers is provided to ensure timely separation of the safety-related buses from offsite power during (sustained) degraded voltage conditions. Two protection schemes are provided. One is provided for undervoltage conditions concurrent with a safety injection (SI) signal while the other is provided for degraded voltage conditions during normal operation (non-SI). Both schemes ensure that loads are not damaged by degraded voltage conditions.

The relevant portion of the 480-Volt AC System consists of Load Centers A, B, C, and D. The system is powered from the 4.16 kV Emergency Safety Feature buses A and B through four step-down transformers.

3.1 Non-SI Situation

The non-SI scheme has four voltage sensing relays on each load center. Channel 1 utilizes one 3271 and one 327T relay; channel 2 does the same. The relays are interconnected in a "one out of two taken twice" channel trip logic such that the logic trips (initiates bus stripping) if degraded voltage is detected by either channel 1 relay (3271 or 327T) concurrently with either channel 2 relay (3271 or 327T). The 3271 relay protects the 480-volt system for degraded voltage over a long duration while the 327T relay protects the system during a large voltage transient for a short duration.

Each 3271 relay has a fixed voltage setpoint and a fixed time delay of approximately 60 seconds. (The voltage setpoint is slightly different for each load center based upon specific loads supplied and length of cable runs.) If voltage remains below setpoint for 60 seconds, the relays initiate bus stripping on that 4kV bus. The 327Ts are inverse time delay relays, and provide protection for more severely degraded voltage of short durations. Each 327T relay has a voltage setpoint range set below the 3271 setpoint voltage. When voltage drops to this setpoint range, the time before relay drop-out will vary inversely with the severity of the voltage drop. At the higher end of the voltage range, the time delay is approximately 11 seconds and at the lower end the time could be as short as 3 seconds. If voltage drops into the setpoint range and does not recover before the time expires, the relays initiate bus stripping on that 4kV bus. A five-position test switch (with positions labeled 12, 11, Normal, 21 and 22) enables the operator to test either relay on either channel. Placing the test switch in any position other than Normal will block the trip signal from its associated load center.

3.2 SI Situation

The scheme for SI has two voltage sensing relays (327Hs) on each load center (one relay per channel, two channels per load center). If both relays on the load center sense an undervoltage condition (along with an SI signal and the associated emergency diesel generator's (EDGs) output breaker being open), they will initiate bus stripping on the associated

4 KV bus. The 327H relays provide a faster response and protection if an SI signal is present during the sustained degraded voltage condition. Each 327H relay has a fixed voltage setpoint and will drop out instantaneously upon undervoltage. If an SI signal is present and the EDG breaker is open, a 10 second timer will start. If voltage does not recover before the 10 seconds expire, bus stripping will initiate. A three-position, spring return to Normal, test switch enables the operator to test either relay. Placing the test switch in any position other than Normal will bypass the trip circuit of both channels and block the trip signal from its associated load center.

4.0 PROPOSED TECHNICAL SPECIFICATION CHANGES

4.1 Changes to TS 3.3.2 - ACTION Statement 18:

The current TS ACTION Statement 18 reads as follows:

"With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the inoperable channel is placed in the tripped condition within 6 hours."

Licensee proposes to revise ACTION statement 18 by adding the following at the end of the statement above.

"Both channels of any one load center may be taken out of service for up to 8 hours in order to perform surveillance testing per Specification 4.3.2.1."

Justification:

Degraded Voltage on 480-Volt Load Center Without a Safety Injection Signal (Non-SI)

As discussed in the system description section above, the Non-SI degraded voltage scheme has four voltage sensing relays on each load center, two relays per channel. The relays are interconnected in a "one out of two taken twice" channel trip logic such that bus stripping is initiated if degraded voltage is detected by either channel 1 logic concurrently with either channel 2 logic.

During surveillance testing, the design of the test switch is such that it would block the trip signal from its associated load center during the duration of a test. Should a degraded condition occur at the load center during testing, the degraded voltage would be sensed on the other load center of the same power train, and the trip signal would initiate sequencer action. Therefore, automatic safety function capability is maintained during the testing of the relays at any load center.

By letter L-92-215, "Request for Additional Information - Proposed License Amendments: 480 Volt Load Centers Degraded Voltage Protection Scheme," dated July 29, 1992, FPL provided the NRC details of the 480 volt load centers degraded protection logic scheme. By the U.S. Nuclear Regulatory Commission letter "Turkey Point Units 3 and 4 - Issuance of Amendments Re: Degraded Voltage Protection Scheme," dated August 20, 1992, the NRC recognized that the undervoltage protection scheme provided appropriate protection logic to ensure the surveillance could be performed.

The design of the undervoltage protection logic has not changed since the issuance of the referenced amendments. The intent of the design is to allow the performance of the TS required monthly surveillance while maintaining the automatic safety function. The requested change to ACTION Statement 18 is consistent with the other ACTION statements in Table 3.3-2, by allowing a load center protection scheme to be inoperable for up to 8 hours for the performance of the required surveillance. Undervoltage on 480-Volt Load Center Coincident With a Safety Injection Signal. As discussed in the system description section above, the scheme for SI has two voltage sensing relays on each load center. If both relays on any of the load center sense an undervoltage condition (along with an SI signal and the associated EDG's output breaker being open), bus stripping will be initiated on the associated 4 KV bus.

During surveillance testing, the trip circuit of both channels is bypassed and blocks the trip signal from its associated load center. Should an undervoltage condition occur at the load center during surveillance testing, the undervoltage would be sensed on the other load center of the same power train and the trip signal would initiate sequencer action. Therefore, automatic safety function capability is maintained during the testing of the relays at any load center. The requested change to ACTION Statement 18 is consistent with the other ACTION statements in Table 3.3-2, by allowing a load center protection scheme to be inoperable for up to 8 hours during the performance of the required surveillance. This is consistent with Allowed Outage Time (AOT) extensions for the Engineered Safety Features Actuation Signals (ESFAS) associated with the Safety Injection system signals and as approved in Reference 1.

4.2 Changes to TS Table 3.3-2, Item 7.b. Functional Unit description

Turkey Point Units 3 and 4 TS Table 3.3-2, "Engineered Safety Features Actuation System Instrumentation," Loss of Power, Item 7.b addresses the requirements for the 480 volt load centers degraded voltage protection coincident with Safety Injection. Table 3.3-2, Item 7.b. Functional Unit description currently reads:

"480V Load Centers 3A, 3B, 3C, 3D, and 4A, 4B, 4C, 4D Degraded Voltage"

Licensee proposes to change the Functional Unit description to read:

"480V Load Centers 3A, 3B, 3C, 3D, and 4A, 4B, 4C, 4D Undervoltage"

4.3 Changes to TS Table 3.3-3, Item 7.b. Functional Unit description

Turkey Point Units 3 and 4 TS Table 3.3-3, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints," Loss of Power, Items 7.b specifies the trip setpoints for the 480 volt load centers degraded voltage protection coincident with Safety Injection. Table 3.3-31 Item 7.b. Functional Unit description currently reads:

"480V Load Centers Degraded voltage"

Licensee proposes to change the Functional Unit description to read:

"480V Load Centers Undervoltage"

4.4 Changes to TS Table 4.3-2, Item 7.b., Functional Unit description

Turkey Point Units 3 and 4 TS Table 4.3-2, "Engineered Safety Features Actuation System Instrumentation Surveillance Requirements," Loss of Power, Items 7.b address the surveillance requirements for the 480-volt load centers degraded voltage protection coincident with Safety Injection. Table 4.3-2, Item 7.b, Functional Unit description currently reads:

"480V Load Centers 3A, 3B, 3C, 3D, and 4A, 4B, 4C, 4D Degraded voltage"

Licensee proposes to change the Functional Unit description to read:

"480V Load Centers 3A, 3B, 3C, 3D, and 4A, 4B, 4C, 4D Undervoltage"

Justification

All of the above proposed changes are administrative in nature and are requested to make the functional unit description consistent with the UFSAR Section 8.2.2.1.1.2, Specifics of the Onsite AC Power System, Subsection "Electric Circuit Protection System Network," and with the 4 kV Bus Loss of Voltage and Bus Stripping Logic Diagram, 5610-TL-1, Sheet 13.

5.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

6.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, and change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (65 FR 56951, dated September 20, 2000). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

Based on the staff evaluation in Sections 3.0 and 4.0 above, the staff concludes that the proposed TS changes are acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there

is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

8.0 REFERENCES

1. Safety Evaluation by the Office of Nuclear Reactor Regulation - "Review of Westinghouse Report," WCAP-10271 Supplement 2 and WCAP-10271 Supplement 2, Revision 1 on "Evaluation of Surveillance Frequencies and Out of Service Times for the Engineered Safety Features Actuation System," February 22, 1989.

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