

Docket No.: 50-387

Mr. Norman W. Curtis  
Vice President  
Engineering and Construction - Nuclear  
Pennsylvania Power & Light Company  
2 North Ninth Street  
Allentown, Pennsylvania 18101

Dear Mr. Curtis:

Subject: Amendment No. 18 to Facility Operating License No. NPF-14 -  
Susquehanna Steam Electric Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 18 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station, Unit 1. The amendment is in response to your letter dated October 7, 1983. This amendment would delete the requirement in Technical Specification Table 3.6.3-1 for the RHR-Shutdown Cooling Valves HV-151F008 and HV-151F009 to isolate on Isolation Signal Z, High Drywell Pressure. This amendment also changes the requirement in Technical Specification Table 3.6.3-1 for the RWCU Suction Valves HV-144F001 and HV-144F004 from isolating on Isolation Signal A, Reactor Vessel Water Level-Low Level (Level 3) to isolating on Isolation Signal B, Reactor Vessel Water Level-Low, Low (Level 2).

A copy of the related safety evaluation supporting Amendment No. 18 to Facility Operating License NPF-14 is enclosed.

Sincerely,

~~Original signed by:~~

A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosures:

- 1. Amendment No. 18 to NPF-14
- 2. Safety Evaluation

cc w/ enclosures:  
See next page

\*SEE PREVIOUS CONCURRENCES

|             |             |             |          |          |
|-------------|-------------|-------------|----------|----------|
| DL:LB#2/PM* | DL:LB#2/LA* | DL:LB#2/BC* | OELD*    | DL:AD/L* |
| RLPerch:pt  | EGHylton    | ASchwencer  | MEWagner | TMNovak  |
| 10/14/83    | 10/14/83    | 10/20/83    | 11/14/83 | 10/27/83 |

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PDR ADDCK 05000387  
P PDR

Susquehanna

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Susquehanna

cc: Governor's Office of State Planning & Development  
Attn: Coordinator, State Clearinghouse  
P O. Box 1323  
Harrisburg, Pennsylvania 17120

Mr. Bruce Thomas, President  
Board of Supervisors  
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Berwick, Pennsylvania 18603

U. S. Environmental Protection Agency  
Attn: EIS Coordinator  
Region III Office  
Curtis Building  
6th and Walnut Streets  
Philadelphia, Pennsylvania 19106



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PENNSYLVANIA POWER AND LIGHT COMPANY  
ALLEGHENY ELECTRIC COOPERATIVE, INC.  
DOCKET NO. 50-387  
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18  
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Pennsylvania Power and Light Company, dated October 7, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment was effective as of October 7, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

**Original signed by:**

A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 18, 1983

*RL*  
DL:LB#2/PM  
RLPerch:pt  
10/14/83

*EGH*  
DL:LB#2/LA  
EGH:ton  
10/14/83

*AS*  
DL:LB#2/BC  
ASchwencer  
10/30/83

*min*  
OELD  
M. Wagner  
10/14/83  
*TD*  
DL:AD/L  
TDNovak  
10/28/83

ATTACHMENT TO LICENSE AMENDMENT NO.  
FACILITY OPERATING LICENSE NO. NPF-14  
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-19  
3/4 6-20

INSERT

3/4 6-19  
3/4 6-20

TABLE 3.6.3-1

PRIMARY CONTAINMENT ISOLATION VALVES

| <u>VALVE FUNCTION AND NUMBER</u>                         | <u>MAXIMUM ISOLATION TIME (Seconds)</u> | <u>ISOLATION SIGNAL(s)<sup>(a)</sup></u> |
|--|---|--|
| <u>a. Automatic Isolation Valves<sup>(b)</sup></u>       |   |  |
| <u>MSIV</u>  |   |  |
| HV-141F022 A,B,C,D                                       | 5                                       | B,C,D,E,P,UA                             |
| HV-141F028 A,B,C,D                                       | 5                                       | B,C,D,E,P,UA                             |
| <u>MSL Drain</u>   |   |  |
| HV-141F016   | 10                                      | B,C,D,E,P,UA                             |
| HV-141F019   | 10                                      | B,C,D,E,P,UA                             |
| <u>RCIC Steam Supply</u>                                 |   |  |
| HV-149F007 <sup>(c)</sup>                                | 20                                      | K  |
| HV-149F008 <sup>(c)</sup>                                | 20                                      | K  |
| HV-149F088   | 3                                       | K  |
| <u>HPCI Steam Supply</u>                                 |   |  |
| HV-155F002 <sup>(c)</sup>                                | 50                                      | L  |
| HV-155F003 <sup>(c)</sup>                                | 50                                      | L  |
| HV-155F100   | 3                                       | L  |
| <u>RHR - Shutdown Cooling Suction<sup>(d)</sup></u>      |   |  |
| HV-151F008   | 52                                      | M,UB                                     |
| HV-151F009   | 52                                      | M,UB                                     |
| <u>RHR - Shutdown Cooling Return/<br/>LPCI Injection</u> |   |  |
| HV-151F122 A,B   | 3                                       | Z  |
| <u>RWCU Suction<sup>(e)</sup></u>                        |   |  |
| HV-144F001   | 30                                      | B,J,W                                    |
| HV-144F004   | 30                                      | B,J,W                                    |
| <u>RHR - Reactor Vessel Head Spray</u>                   |   |  |
| HV-151F022   | 30                                      | M,UB,Z                                   |
| HV-151F023   | 20                                      | M,UB,Z                                   |

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

| <u>VALVE FUNCTION AND NUMBER</u>                            | <u>MAXIMUM ISOLATION TIME (Seconds)</u> | <u>ISOLATION SIGNAL(S)<sup>(a)</sup></u> |
|---|---|--|
| <u>Automatic Isolation Valves<sup>(b)</sup> (Continued)</u> |   |  |
| <u>Containment Instrument Gas</u>                           |   |  |
| HV-12603  | 20                                      | X  |
| SV-12605  | N/A                                     | X  |
| SV-12651  | N/A                                     | X  |
| SV-12661  | N/A                                     | Y  |
| SV-12671  | N/A                                     | Y  |
| <u>RBCCW</u>  |   |  |
| HV-11313  | 30                                      | X  |
| HV-11314  | 30                                      | X  |
| HV-11345  | 30                                      | X  |
| HV-11346  | 30                                      | X  |
| <u>Containment Purge</u>                                    |   |  |
| HV-15703  | 19                                      | Y,R                                      |
| HV-15704  | 19                                      | Y,R                                      |
| HV-15705  | 5                                       | Y,R                                      |
| HV-15711  | 5                                       | Y,R                                      |
| HV-15713  | 30                                      | Y,R                                      |
| HV-15714  | 30                                      | Y,R                                      |
| HV-15721  | 6                                       | Y,R                                      |
| HV-15722  | 30                                      | Y,R                                      |
| HV-15723  | 30                                      | Y,R                                      |
| HV-15724  | 19                                      | Y,R                                      |
| HV-15725  | 19                                      | Y,R                                      |
| <u>RHR - Drywell Spray<sup>(f)</sup></u>                    |   |  |
| HV-151F016 A,B  | 90                                      | G  |
| <u>RB Chilled Water</u>                                     |   |  |
| HV-18781 A1,A2,B1,B2  | 40                                      | X  |
| HV-18782 A1,A2,B1,B2  | 6                                       | X  |
| HV-18791 A1,A2,B1,B2  | 15-                                     | Y  |
| HV-18792 A1,A2,B1,B2  | 4                                       | Y  |



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION  
AMENDMENT NO. 18 TO NPF-14  
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1  
DOCKET NO. 50-387

Introduction

By letter dated October 7, 1983, the licensee proposed an emergency Technical Specification change regarding the operability of (1) the RHR-Shutdown Cooling Valves HV-151F008 and HV-151F009 and (2) the RWCU Suction Valves HV-144F001 and HV-144F004 as required in Table 3.6.3-1 (Primary Containment Isolation Valves) of Technical Specification 3.6.3. The proposed change would delete the requirement in the table for the RHR-Shutdown Cooling Valves HV-151F008 and HV-151F009 to isolate on Isolation Signal 2, High Drywell Pressure. The proposed change would also change the requirement in the table for the RWCU Suction Valves HV-144F001 and HV-144F004 from isolating on Isolation Signal A, Reactor Vessel Water Level-Low (Level 3) to isolating on Isolation Signal B, Reactor Vessel Water Level-Low, Low (Level 2). The proposed change would delete these requirements as typographical errors in the table.

Evaluation

Technical Specification 3.6.3 addresses the operability of the primary containment isolation valves and Table 3.6.3-1, of Technical Specification 3.6.3, lists the primary containment isolation valves by valve number and function, and lists their respective maximum isolation times and isolation signals. The first proposed specification change is to delete the requirement for the RHR-Shutdown Cooling Valves HV-151F008 and HV-151F009 to isolate on Isolation Signal Z, High Drywell Pressure. This proposed specification change corrects a typographical error. The present system design does not include this provision for Isolation Signal Z and was never intended to include this requirement. The present design with the RHR-Shutdown Cooling valves not isolating on a High Drywell Pressure signal is consistent with all the recently licensed BWRs and with the GE Standard Technical Specification. In addition, Table 3.3.2-1, Isolation Actuation Instrumentation, of the licensee's Technical Specification does not require the RHR System Shutdown Cooling Mode Isolation to include an isolation on High Drywell Pressure.

The second proposed specification change is to modify the requirement for the RWCU Suction Valves HV-144F001 and HV-144F004 from isolating on Isolation Signal A, Reactor Vessel Water Level-Low (Level 3) to isolating on Isolation Signal B, Reactor Vessel Water Level-Low, Low (Level 2). This proposed specification change also corrects a typographical error. The present design is for the RWCU Suction valves to isolate on Isolation Signal B, Reactor Vessel Water Level-Low, Low (Level 2). This design is consistent with all of the recently licensed BWRs and with the GE Standard Technical Specifications. In addition, Table 3.3.2-1 Isolation Actuation Instrumentation, of the licensee's Technical Specification requires the RWCU Suction valves to isolate on a Reactor Vessel Water Level-Low, Low (Level 2).

The NRC staff has made a final determination that the proposed changes involve no significant hazards consideration. The changes are a result of typographical errors. A purely administrative change to Technical Specifications - for example, a change to achieve consistency throughout the Technical Specifications, correction of an error, or a change in nomenclature is an example of a change found to involve no significant hazards consideration. The administrative change (example (i) of 48 FR 14871) applies to the proposed changes; therefore, the proposed changes involve no significant hazards consideration. Licensee meets the test of prompt notification. On October 7, 1983, the need for the correction was identified, and on the same day the request for change was transmitted by letter (sent via telecopy facilities) to the NRC staff.

#### Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) this amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, creation of a new or different accident from any previously evaluated, or a significant reduction in a safety margin and, therefore, does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 7, 1983

\*SEE PREVIOUS CONCURRENCES

|             |             |          |
|-------------|-------------|----------|
| DL:LB#2/PM* | DL:LB#2/BC* | DL:SSPB* |
| RLPerch:pt  | ASchwencer  | DHoffman |
| 10/14/83    | 10/20/83    | 10/21/83 |

DATED: November 18, 1983

AMENDMENT No. 18 TO NPF-14 (Susquehanna)

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