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Mr. Norman W. Curtis
Vice President
Engineering and Construction - Nuclear
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Dear Mr. Curtis:

Subject: Amendment No. 8 to Facility Operating License No. NPF-14 -
Susquehanna Steam Electric Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 8 to Facility Operating License No. NPF-14 for the Susquehanna Steam Electric Station, Unit 1. The amendment is in response to your letter dated October 22, 1982. This amendment makes corrections to Technical Specifications 4.6.1.2.h.; 3.9.3.1; and 4.11.2.7.1. These changes are administrative in nature and serve to correct typographical errors.

A copy of the related safety evaluation supporting Amendment No. 8 to Facility Operating License NPF-14 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:

1. Amendment No. 8 to NPF-14
2. Safety Evaluation
3. Federal Register Notice

cc w/ enclosures:
See next page

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Legal objection

OFFICE	DL:LB#2/PM RPerch:pt	DL:LB#2/LA EHylton	OELD Cutchin	DL:LB#2/BC ASchwencer	DL:AD/L TNovak		
SURNAME							
DATE	1/26/83	2/7/83	1/24/83	2/7/83	2/7/83		

Susquehanna

Mr. Norman W. Curtis
Vice President
Engineering and Construction
Pennsylvania Power & Light Company
Allentown, Pennsylvania 18101

ccs: Jay Silberg, Esquire
Shaw, Pittman, Potts & Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Edward M. Nagel, Esquire
General Counsel and Secretary
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. William E. Barberich
Nuclear Licensing Group Supervisor
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. G. Rhodes
Resident Inspector
P. O. Box 52
Shickshinny, Pennsylvania 18655

Gerald R. Schultz, Esquire
Susquehanna Environmental Advocates
P. O. Box 1560
Wilkes-Barre, Pennsylvania 18703

Mr. E. B. Poser
Project Engineer
Bechtel Power Corporation
P. O. Box 3965
San Francisco, California 94119

Dr. Judith H. Johnsrud
Co-Director
Environmental Coalition on Nuclear Power
433 Orlando Avenue
State College, Pennsylvania 16801

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection Resources
Commonwealth of Pennsylvania
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Ms. Colleen Marsh
P. O. Box 538A, RD #4
Mountain Top, Pennsylvania 18707

Mr. Thomas J. Halligan
Correspondent
The Citizens Against Nuclear Dangers
P. O. Box 5
Scranton, Pennsylvania 18501

Mr. J. W. Millard
Project Manager
Mail Code 395
General Electric Company
175 Curtner Avenue
San Jose, California 95125

Robert W. Adler, Esquire
Office of Attorney General
505 Executive House
P. O. Box 2357
Harrisburg, Pennsylvania 17120

PENNSYLVANIA POWER & LIGHT COMPANY
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8
 License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for an amendment filed by the Pennsylvania Power & Light Company dated October 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 8, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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 PDR ADOCK 05000387
 PDR

OFFICE
SURNAME
DATE

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DL:AD/L	TNovak	2/1/83

Handwritten notes:
 1. Legal Dept. check
 2. Review of documents
 3. Resubmission

Attachment:
 Changes to the Technical Specifications
 Date of Issuance: FEB 7 1983

A. Schwencer, Chief
 Licensing Branch No. 2
 Division of Licensing

FOR THE NUCLEAR REGULATORY COMMISSION

3. This amendment is effective as of the date of issuance.

ATTACHMENT TO LICENSE AMENDMENT NO. 8
FACILITY OPERATING LICENSE NO. NPF-14
DOCKET NO. 50-387

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-3
3/4 6-4

3/4 8-17
3/4 8-18

3/4 11-17
3/4 11-18

INSERT

3/4 6-3
3/4 6-4

3/4 8-17
3/4 8-18

3/4 11-17
3/4 11-18

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

restore:

- a. The overall integrated leakage rate to less than or equal to $0.75 L_a$, and
- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, main steam line drain valves* and valves which are hydrostatically leak tested per Table 3.6.3-1, subject to Type B and C tests to less than or equal to $0.60 L_a$, and
- c. The leakage rate to less than or equal to 46 scf per hour for all four main steam lines through the isolation valves, and
- d. The leakage rate to less than or equal to 1.2 scf per hour for any one main steam line drain valve, and
- e. The combined leakage rate for all containment isolation valves in hydrostatically tested lines which penetrate the primary containment to less than or equal to 3.3 gpm,

prior to increasing reactor coolant system temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P_a , 45.0 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
- b. If any periodic Type A test fails to meet $.75 L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $.75 L_a$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $.75 L_a$, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_a$.
 2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P_a , 45.0 psig.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. Type B and C tests shall be conducted with gas at P_a , 45.0 psig,* at intervals no greater than 24 months except for tests involving:
1. Air locks,
 2. Main steam line isolation valves and main steam line drain valves,
 3. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 4. Purge supply and exhaust isolation valves with resilient material seals.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves and main steam line drain valves shall be leak tested at least once per 18 months.
- g. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at least once per 18 months.
- h. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.8.2.
- i. The provisions of Specification 4.0.2 are not applicable to 24 month or 40 ± 10 month surveillance intervals.

*Unless a hydraulic test is required per Table 3.6.3-1.

ELECTRICAL POWER SYSTEMS

3/4.8.3 ONSITE POWER DISTRIBUTION SYSTEMS

DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.3.1 The following power distribution system divisions shall be energized with tie breakers open both between redundant buses within the unit and between units at the same station:

a. A.C. power distribution:

1. Division I, consisting of:

- a) Load group Channel "A", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A201
 - 2) 480 volt A.C. load center 1B210
 - 3) 480 volt A.C. motor control center 0B516
- b) Load group Channel "C", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A203
 - 2) 480 volt A.C. load center 1B230
 - 3) 480 volt A.C. motor control center 0B536
- c) Load group 480 volt A.C. motor control centers 0B517, 0B136
1B216, 1B236
1B217, 1B237
1B219
- d) Load group 208/120 volt A.C. instrument panels 1Y216, 1Y236

2. Division II, consisting of:

- a) Load group Channel "B", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A202
 - 2) 480 volt A.C. load center 1B220
 - 3) 480 volt A.C. motor control center 0B526
- b) Load group Channel "D", consisting of:
 - 1) 4160 volt A.C. switchgear bus 1A204
 - 2) 480 volt A.C. load center 1B240
 - 3) 480 volt A.C. motor control center 0B546
- c) Load group 480 volt A.C. motor control centers 0B527, 0B146
1B226, 1B246
1B227, 1B247
1B229
- d) Load group 208/120 volt A.C. instrument panels 1Y226, 1Y246

b. D.C. power distribution:

1. Division I, consisting of:

- a) Load group Channel "A", consisting of:
 - 1) 125 volt DC buses 1D612, 1D614
 - 2) Fuse box 1D611
- b) Load group Channel "C", consisting of:
 - 1) 125 volt DC buses 1D632, 1D634
 - 2) Fuse box 1D631
- c) Load group "I", consisting of:
 - 1) 250 volt DC buses 1D652, 1D254
 - 2) Fuse box 1D651
- d) Load group "I", consisting of:
 - 1) ± 24 volt DC buses 1D672
 - 2) Fuse box 1D671

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

D.C. power distribution: (Continued)

2. Division II, consisting of:

- a) Load group Channel "B" consisting of:
 - 1) 125 volt DC buses 1D622, 1D624
 - 2) Fuse box 1D621

- b) Load group Channel "D" consisting of:
 - 1) 125 volt DC buses 1D642, 1D644
 - 2) Fuse box 1D641

- c) Load group "II" consisting of:
 - 1) 250 volt DC buses 1D662, 1D264, 1D274
 - 2) Fuse box 1D661

- d) Load group "II" consisting of:
 - 1) ± 24 volt DC buses 1D682
 - 2) Fuse box 1D681

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With one of the above required A.C. distribution system load groups not energized, re-energize the load group within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

- b. With one of the above required D.C. distribution system load groups not energized, re-energize the load group within 2 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.8.3.1 Each of the above required power distribution system load groups shall be determined energized at least once per 7 days by verifying correct breaker alignment and voltage on the busses/MCCs/panels.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

3.11.2.6 The concentration of hydrogen or oxygen in the main condenser offgas treatment system shall be limited to less than or equal to 4% by volume.

APPLICABILITY: Whenever the main condenser air ejector (evacuation) system is in operation.

ACTION:

- a. With the concentration of hydrogen in the main condenser offgas treatment system exceeding the limit, restore the concentration to within the limit within 48 hours.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.6 The concentration of hydrogen in the main condenser offgas treatment system shall be determined to be within the above limits by continuously monitoring the waste gases in the main-condenser offgas treatment system with the hydrogen monitors required OPERABLE by Table 3.3.7.11-1 of Specification 3.3.7.11.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

LIMITING CONDITION FOR OPERATION

3.11.2.7 The radioactivity rate of the noble gases Kr-85m, Kr-87, Kr-88, Xe-133, Xe-135, and Xe-138 measured at the motive steam jet condenser discharge shall be limited to less than or equal to 330 millicuries/second.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2* and 3*.

ACTION:

With the gross radioactivity rate of the specified noble gases at the motive steam jet condenser discharge exceeding 330 millicuries/second, restore the gross radioactivity rate to within its limit within 72 hours or be in at least HOT STANDBY within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.11.2.7.1 The radioactivity rate of noble gases at the motive steam jet condenser discharge shall be continuously monitored in accordance with Specification 3.3.7.11.

4.11.2.7.2 The gross radioactivity rate of the specified noble gases from the motive steam jet condenser discharge shall be determined to be within the limits of Specification 3.11.2.7 at the following frequencies by performing an isotopic analysis of a representative sample of gases taken at the discharge:

- a. At least once per 31 days.
- b. Within 4 hours following an increase, as indicated by the Condenser Offgas Pre-Treatment Radioactivity Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady state fission gas release from the primary coolant.

*When the main condenser air ejector is in operation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION
AMENDMENT NO. 8 TO NPF-14
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-387

Introduction

The licensee in a letter dated October 22, 1982, proposed changes to the Technical Specifications of the operating license for Susquehanna Steam Electric Station, Unit 1 which are as follows:

- a) In Technical Specification 4.6.1.2.h, change the end of the sentence from "per Surveillance Requirement 4.6.1.8.3." to read "per Surveillance Requirement 4.6.1.8.2."
- b) In Technical Specification 3.8.3.1.b.2.c)1), change load group "1D622" to read "1D662".
- c) In Action Statement b) of Technical Specification 3.8.3.1, add to the end of the statement "24 hours."
- d) In Technical Specification 4.11.2.7.1, change the end of the sentence from "Specification 3.3.7.12." to read "Specification 3.3.7.11."

Evaluation

Each of the above changes proposed by the licensee is as a result of typographical error in the Technical Specifications. These changes are administrative in nature and serve to clarify the Technical Specifications effected. On this basis the above changes are found acceptable to the staff.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: FEB 7 1983

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-387

PENNSYLVANIA POWER & LIGHT COMPANY

ALLEGHENY ELECTRIC COOPERATIVE, INC.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 8 to Facility Operating License No. NPF-14, issued to Pennsylvania Power & Light Company and Allegheny Electric Cooperative, Inc., for Susquehanna Steam Electric Station, Unit 1 (the facility) located in Luzerne County, Pennsylvania. This amendment grants changes to Technical Specifications to correct typographical errors noted in the document. The changes are administrative in nature. This amendment is effective as of the date of issuance.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) and environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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PDR ADOCK 05000387
P PDR

OFFICE ▶
SURNAME ▶
DATE ▶

For further details with respect to this action, see (1) the application for the amendment dated October 22, 1982; (2) Amendment No. 8 to License NPF-14 dated February 7, 1983 ; and (3) the Commission's evaluation dated February 7, 1983. All of these items are available for public inspection at the Commission's Public Document Room 1717 H Street, N. W., Washington, D. C. 20555, and at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Milkes-Barre, Pennsylvania 18701. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

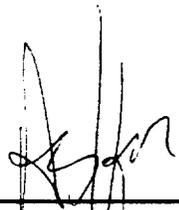
Dated at Bethesda, Maryland, this 7th day of February 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

*Original reference to
file of R. notice*



OFFICE	DL:LB#2/RM	DL:LB#2/LA	OELD	DL:LB#2/BC	DL:AD/L		
SURNAME	RPerch:pt	EL	LUTCHIN	ASchwencer	TNovak		
DATE	1/26/83	2/7/83	1/26/83	2/7/83	2/7/83		