

October 12, 1999

Mr. James Knubel  
Chief Nuclear Officer  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING  
UNIT NO. 3 RE: SAFETY INJECTION PUMP ALLOWED OUTAGE TIME (TAC  
NO. MA5360)

Dear Mr. Knubel:

The Commission has issued the enclosed Amendment No. 196 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3 (IP3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated April 9, 1999. The amendment increases the allowable outage time for one safety injection pump from 24 to 72 hours.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:  
George F. Wunder, Project Manager, Section 1  
Project Directorate 1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No.196 to DPR-64  
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See next page

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DATED: October 12, 1999

AMENDMENT NO. 196 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT  
UNIT 3

Docket File  
PUBLIC  
PDI Reading  
E. Adensam (e-mail)  
S. Bajwa  
S. Little  
G. Wunder  
L. Berry  
OGC  
G. Hill (2), T-5 C3  
W. Beckner  
ACRS  
M. Opredek, Region I  
R. Scholl (e-mail SE only)

cc: Plant Service list

DATED: October 12, 1999

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 12, 1999

Mr. James Knubel  
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George F. Wunder, Project Manager, Section 1  
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Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 196 to DPR-64  
2. Safety Evaluation

cc w/encls: See next page

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Unit No. 3

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2909 29<sup>th</sup> St. NW  
Washington, DC 20008-3416



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 196  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 9, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 196 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read 'S. Singh Bajwa', written over a horizontal line.

S. Singh Bajwa, Chief, Section 1  
Project Directorate 1  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical Specifications

Date of Issuance: October 12, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 196

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contain marginal lines indicating the areas of change.

Remove Page  
3.3-4

Insert Page  
3.3-4

- a. The accumulators may be isolated during the performance of the reactor coolant system hydrostatic tests.

For the purpose of accumulator check valve leakage testing, one accumulator may be isolated at a time, for up to 8 hours, provided the reactor is in the hot shutdown condition.

- b. One safety injection pump may be out of service, provided the pump is restored to an operable status within 72 hours.
- c. One residual heat removal pump may be out of service, provided the pump is restored to an operable status within 24 hours.
- d. One residual heat exchanger may be out of service provided that it is restored to an operable status within 48 hours.
- e. Any valve required for the functioning of the system during and following accident conditions may be inoperable provided that it is restored to an operable status within 24 hours and all valves in the system that provide the duplicate function are operable.
- f. DELETED
- g. One refueling water storage tank low level alarm may be inoperable for up to 7 days provided the other low level alarm is operable.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 196 TO FACILITY OPERATING LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated April 9, 1999, the Power Authority of the State of New York (the licensee) proposed a change to the Technical Specifications (TSs) for the Indian Point 3 Nuclear Generating Unit No. 3. The proposed TS change will increase the allowed outage time (AOT) for any one safety injection pump from 24 hours to 72 hours. The licensee submitted its TS Improved Standard Technical Specification (ISTS) conversion application in December 1998. The ISTS conversion application is currently under staff review. In the interim, this line item improvement change is requested by the licensee because the 24-hour AOT is restrictive and could result in a plant shutdown to support future corrective maintenance.

2.0 EVALUATION

The emergency core cooling system (ECCS) at IP3 consists of three systems: high head safety injection (HHSI), residual heat removal (RHR) and containment recirculation. The HHSI system is divided into three 50 percent capacity subsystems, each consisting of a one safety injection pump. The RHR system is divided into two 100 percent capacity subsystems. Containment Recirculation is divided into two 100 percent capacity subsystems. The three ECCS systems are grouped into three trains such that any two of the three trains are capable of meeting all ECCS capacity assumed in the accident analyses. Because of the redundancy of the system, the HHSI remains capable of providing 100 percent flow capacity with one safety injection pump inoperable. Since the single failure criterion is relaxed during the time period covered by the TS allowed outage time, this TS change will not impact the ECCS requirement that at least 100 percent of the ECCS flow rate equivalent to operable ECCS trains be available.

TS 3.5.2 ECCS - Operating condition A (one or more trains are inoperable, restore train(s) to operable status within 72 hours) is consistent with the approved NUREG-1431, "Standard Technical Specifications - Westinghouse Plants," Revision 1, dated April 1995. The change in allowable outage time from 24 to 72 hours does not require physical changes to plant systems, components or operation and also does not affect how the safety injection pumps are operating under a design basis accident. Furthermore, it will facilitate the future corrective maintenance activities and may prevent a plant shutdown. The staff, therefore, finds this change acceptable.

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### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (64 FR 29713). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Desai

Date: October 12, 1999