

December 8, 1994

Mr. William J. Cahill, Jr.
Executive Vice President - Nuclear
Generation
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION
DETERMINATION, AND OPPORTUNITY FOR A HEARING, INDIAN POINT
NUCLEAR GENERATING UNIT NO. 3 (TAC NO. M91047)

Dear Mr. Cahill:

The Commission has requested the Office of the Federal Register to publish the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazard Consideration Determination, and Opportunity for a Hearing." This notice relates to your application for amendment dated December 8, 1994, which would revise Section 4.4 of the Indian Point Nuclear Generating Unit No. 3 Technical Specifications. Specifically, revision of TS 4.4.E.1 would allow a one-time extension to the 30-month interval requirement for leak rate testing of Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. The leak rate test for these valves would be performed prior to T_{avg} exceeding 350°F.

Sincerely,

Original signed by

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: Notice of Consideration
cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Executive Vice President - Nuclear
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Power Authority of the State of New York
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Sincerely,

A handwritten signature in cursive script, appearing to read "N. F. Conicella".

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: Notice of Consideration

cc w/encl: See next page

William J. Cahill, Jr.
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector
Indian Point 3 Nuclear Power Plant
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

Mr. Gerald C. Goldstein
Assistant General Counsel
Power Authority of the State
of New York
1633 Broadway
New York, NY 10019

Mr. Charles W. Jackson
Manager, Nuclear Safety and
Licensing
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, NY 10511

Mr. Robert G. Schoenberger
First Executive Vice President
and Chief Operating Officer
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Mr. Leslie M. Hill
Resident Manager
Indian Point 3 Nuclear Power Plant
P.O. Box 215
Buchanan, NY 10511

Mr. Richard L. Patch, Director
Quality Assurance
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Ms. Charlene D. Faison
Director Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Union of Concerned Scientists
Attn: Mr. Robert D. Pollard
1616 P Street, NW, Suite 310
Washington, DC 20036

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, NY 12223

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

UNITED STATES NUCLEAR REGULATORY COMMISSION

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendment to Facility Operating License No. DRP-64, issued to the Power Authority of the State of New York (the licensee), for operation of Indian Point Nuclear Generating Unit No. 3 (Indian Point 3) located in Westchester County, New York.

The proposed amendment would revise Section 4.4 of the Indian Point 3 Technical Specifications. Specifically, TS 4.4.E.1 would be revised to allow a one-time extension to the 30-month interval requirement for leak rate testing of Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. This one-time extension for leak rate testing of the RHR valves would be deferred until prior to return to power following the current outage, which is defined as prior to T_{avg} exceeding 350 °F.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facilities in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Consistent with the criteria of 10 CFR 50.92, the enclosed application is judged to involve no significant hazards based on the following information:

1. Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response:

The proposed license amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed change is limited to a one-time extension of the containment isolation valve leak rate test for RHR valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. Testing the RHR containment isolation valves while T_{avg} is above 200 °F and below 350 °F is considered to be a more practical time to test the valves because, in this configuration, the reactor coolant pumps and steam generators can be used to remove decay heat. Additionally, the revised testing procedure required to test the RHR containment isolation valves with T_{avg} above 200 °F and below 350 °F can be performed in accordance with the current Technical Specifications. Therefore, the probability of a previously evaluated accident is not significantly increased. The consequences of a previously evaluated accident would not be significantly increased because each of the three RHR lines associated with valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 has redundant isolation barriers and is supplied by the IVSWS [isolation valve seal water system] which would minimize any leakage past the isolation barriers. Further, due to the periodic

surveillance that ensures that leakage from RHR components located outside containment does not exceed two gallons per hour, even if significant leakage past the RHR containment isolation valves occurred, this would not significantly affect off-site exposures.

2. Does the proposed license amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response:

The proposed license amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed change does not introduce new accident initiators or failure mechanisms since the change does not alter the physical characteristics of any plant system or component. The change is limited to a one-time extension to the leak rate test interval for RHR valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870.

3. Does the proposed amendment involve a significant reduction in a margin of safety?

Response:

The proposed amendment does not involve a significant reduction in a margin of safety. Testing the RHR containment isolation valves while T_{avg} is above 200 °F and below 350 °F is considered to be a more practical time to test the valves because, in this configuration, the reactor coolant pumps and steam generators can be used to remove decay heat. Additionally, the revised testing procedure required to test the RHR containment isolation valves with T_{avg} above 200 °F and below 350 °F can be performed in accordance with the current Technical Specifications. Therefore, there is not a significant reduction in a margin of safety. With respect to containment integrity, there is not a significant reduction in a margin of safety because each of the three RHR lines associated with valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 has redundant isolation barriers and is supplied by the IVSWS which would minimize any leakage past the isolation barriers. The most recent test results associated with valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 show that the leak rates of the valves were well within the acceptance criteria of the tests. Further, due to the periodic surveillance that ensures that leakage from RHR components located outside containment does not exceed two gallons per hour, even if significant leakage past the RHR containment isolation valves occurred, this would not significantly affect off-site exposures.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facilities, the Commission may issue the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville Maryland,

from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By January 12, 1995, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the White Plains Public Library, 100 Martine Avenue, White Plains, New York 10601. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The

petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. The petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or

fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last 10 days of the notice period, it is

requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Michael J. Case: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Mr. Charles M. Pratt, 10 Columbus Circle, New York, New York 10019, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated December 8, 1994, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street,

NW., Washington, DC 20555 and at the local public document room located at the White Plains Public Library, 100 Martine Avenue, White Plains, New York 10601.

Dated at Rockville, Maryland, this 8th day of December 1994.

FOR THE NUCLEAR REGULATORY COMMISSION

Donald S. Brinkman

Donald S. Brinkman, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation