

January 13, 1995

Mr. William J. Cahill, Jr.
Executive Vice President, Nuclear
Generation
Power Authority of the State of
New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
(TAC NO. M91047)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 158 to Facility Operating License No. DPR-64 for Indian Point Nuclear Generating Unit No. 3. The amendment consists of change to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 8, 1994.

The amendment revises TS Section 4.4.E.1 to allow a one-time extension to the 30-month interval requirement for leak rate testing of Residual Heat Removal containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744 and AC-MOV-1870. Specifically, the local leak rate test (LLRT) interval for these valves will be extended until the return to power following the current outage, which is defined as prior to T_{avg} exceeding 350 °F.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 158 to DPR-64
2. Safety Evaluation

cc w/encls: See next page

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PDR ADOCK 05000286
P PDR

*See previous concurrence

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| NAME | CVogan | | JHarold:cn | | NConicella | | RBarrett | eHollen |
| DATE | 01/4/95 | | 01/4/95 | | 01/4/95 | | 01/03/95 | 01/12/95 |
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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 13, 1995

Mr. William J. Cahill, Jr.
Executive Vice President, Nuclear
Generation
Power Authority of the State of
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123 Main Street
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Sincerely,

A handwritten signature in cursive script, appearing to read "N. F. Conicella".

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 158 to DPR-64
2. Safety Evaluation

cc w/encls: See next page

William J. Cahill, Jr.
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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Union of Concerned Scientists
Attn: Mr. Robert D. Pollard
1616 P Street, NW, Suite 310
Washington, DC 20036

DATED: January 13, 1995

AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT UNIT 3

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

J. Zwolinski, 14/H/3

Michael J. Case

C. Vogan

N. Conicella

OGC

D. Hagan, T-4 A43

G. Hill (2), T-5 C3

C. Grimes, 11/E/22

R. Barrett

ACRS (4)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

cc: Plant Service list

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 158
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated December 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 158, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael J. Case, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 13, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 158

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Page

4.4-4

Insert Page

4.4-4

E. Containment Isolation Valves

1. Tests and Frequency

- a. Isolation valves in Table 4.4-1 shall be tested for operability at intervals no greater than 30 months (24 months + 25%).
- b. Isolation valves in Table 4.4-1 which are pressurized by the Weld Channel and Penetration Pressurization System shall be leakage tested as part of the Weld Channel and Penetration Pressurization System Test at intervals no greater than 30 months (24 months + 25%).
- c. Isolation valves in Table 4.4-1 which are pressurized by the Isolation Valve Seal Water System shall be tested at intervals no greater than 30 months (24 months + 25%)* as part of an overall Isolation Valve Seal Water System Test.
- d. Isolation valves in Table 4.4-1 which are not pressurized will be tested at intervals no greater than 30 months (24 months + 25%)*.
- e. Isolation valves in Table 4.4-1 shall be tested with the medium and at the pressure specified therein.

2. Acceptance Criteria

- a. The combined leakage rate for the following shall be less than $0.5 L_a$: isolation valves listed in Table 4.4-1 subject to gas or nitrogen pressurization testing, air lock testing as specified in D.1, portions of the sensitive leakage rate test described in C.1 which pertain to containment penetrations and double-gasketed seals.
- b. The leakage rate into containment for the isolation valves sealed with the service water system is 0.36 gpm per fan cooler.
- c. The leakage rate for the Isolation Valve Seal Water System shall not exceed 14,700 cc/hr.

* During the Restart and Continuous Improvement Outage, leakage testing of Containment Isolation Valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 may exceed the 30 month interval requirement provided the valves are tested prior to T_{avg} exceeding 350°F.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated December 8, 1994, the Power Authority of the State of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3) Technical Specifications (TSs). The requested changes would revise TS section 4.4.E.1 to allow a one-time extension to the 30-month interval requirement for leak rate testing of Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. Specifically, the local leak rate test (LLRT) interval for these valves will be extended until the return to power following the current outage, which is defined as prior to T_{avg} exceeding 350 °F.

3.0 EVALUATION

The licensee commenced operating on a 24-month fuel cycle, instead of the previous 18-month fuel cycle, starting with fuel cycle 9. Fuel cycle 9 started in August 1992. The requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.3, indicate that Type C LLRTs must be performed during each reactor shutdown for refueling at intervals no greater than 2 years (24 months). On February 19, 1993, the NRC staff issued the Appendix J Exemption and on April 9, 1993, the NRC staff issued the TS Amendment that allowed Type C local leak rate tests (LLRTs) to be performed at intervals up to 30 months, thus, permitting operation on a 24-month fuel cycle.

Approximately six months after startup from the 8/9 refueling outage, IP3 began an extended unplanned non-refueling outage and is expected to restart from this current outage in early 1995. In November and December of 1994, RHR containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 became due for their Type C LLRTs. Currently, the interval for Type C testing of these valves is 30 months, as prescribed in TS 4.4.E.1. These LLRTs are normally performed during a refueling outage when the reactor is defueled and the RHR system is not providing a source of cooling water. The current outage is a nonrefueling outage; therefore, the reactor is not defueled and the RHR system is providing core cooling water. The licensee has requested a one-time change to TS 4.4.E.1 to allow Type C LLRTs of the above listed valves to be deferred until the return to power following the current outage, which is defined as prior to T_{avg} exceeding 350 °F.

When T_{avg} is above 200 °F and below 350 °F, the RHR system may be removed from service to perform the LLRTs. Above 200 °F the reactor coolant pumps and steam generators can be used to remove reactor decay heat. This configuration provides a diverse and redundant method to remove decay heat.

The NRC staff has reviewed the LLRT data provided by the licensee as well as the methodology used by the licensee to extrapolate LLRT data to a longer test interval and the NRC staff concludes that there is reasonable assurance that the containment leakage rate would be maintained within acceptable limits with a one-time LLRT interval increase until the return to power following the current outage, which is defined as prior to T_{avg} exceeding 350 °F. The prerequisite exemption from the testing interval requirement of Appendix J to 10 CFR Part 50 was issued on November 4, 1994.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 64223). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Harold

Date: January 13, 1995