

MEMORANDUM TO: File

FROM: Peter W. Eselgroth, Chief /RA/  
Projects Branch 2  
Division of Reactor Projects

DATE: December 22, 2000

SUBJECT: NOVEMBER 16, 2000 PUBLIC MEETING SUMMARY -  
INDIAN POINT 2 RECENT INSPECTION RESULTS AND  
PLANT STATUS

On November 16, 2000, the Nuclear Regulatory Commission (NRC) conducted a public meeting to discuss the results of recent NRC inspections; to provide a status of the NRC's ongoing review of Indian Point 2's steam generator replacement activities; and to describe the future NRC activities to be conducted at Indian Point 2 (See Attachment 1, November 16, 2000 Public Meeting Presentation Material). The meeting, was held at 7 p.m., in the Cortlandt Town Hall in Cortlandt Manor, N.Y. The meeting was attended by local officials and members of the public. The NRC provided an informational forum, which allowed for questions to be asked by the attendees. During the course of the question and answer session, the NRC stated it would provide more complete answers to certain questions, and post them in the meeting summary on the NRC Indian Point 2 web page. These questions and answers are provided below:

**Q. How do the Strontium-90 (Sr-90) levels (release, environmental levels) for IP2 compare with the rest of the United States?**

**A.** Sr 90 is routinely released from nuclear power stations in very small amounts. A typical airborne release from a Pressurized Water Reactor (PWR) or a Boiling Water Reactor (BWR) is in the order of less than 100 microcuries per year for airborne release and less than 10 millicuries per year for liquid effluents. Dose to members of the public from this level of release is typically a fraction of a millirem.

Indian Point 2's 1999 Annual Effluent and Waste Disposal Report, dated April 28, 2000 reported an airborne release of 28.4 microcuries and a liquid effluent release of 2.33 millicuries. These values are below the typical releases expected for a PWR or BWR.

Generic improvement in fuel performance over the last several years has either reduced or has had the potential to reduce the amount of radioactive material released in effluents from most plants. In addition, at BWRs, the reduction in the amount of airborne radioactive materials being released at some plants since the early and mid-1970s' is due in large part to the installation of the Augmented Off Gas (AOG) systems, many of which were required to be installed to meet the provisions of Appendix I to 10 CFR part 50." At no time, has doses to the public exceeded regulatory limits.

**Q. How can one review a copy of the Office of Inspector General's (OIG) report "NRC's Response to February 15, 2000, Steam Generator Tube Rupture at Indian Point Unit 2 Power Plant"?**

**A.** The OIG report is publicly available on the NRC web-site at:  
<http://www.nrc.gov/NRC/OIG/index.html>

**Q.** **How many public meetings are held under the new program as compared to the old program?**

**A.** The type and number of public meetings to discuss licensee performance issues is essentially unchanged from the previous oversight process to the new Reactor Oversight Process (ROP). Under both processes the number of meetings held and the level of NRC and licensee management involvement is directly related to the number and the severity of performance issues for each plant. For good performing plants, the number of public meetings to discuss licensee performance has actually increased under the ROP. Under the previous oversight process, a good performing plant might have had a meeting to review the NRC's assessment of licensee performance only once every two years. Under the new ROP, all plants regardless of performance, have an annual meeting to review the NRC's assessment results.

**Q.** **How will the public know when Indian Point 2 is removed from the "agency focus" status? How can that information be accessed on the NRC web page?**

**A.** The term "agency focus" was designated by Senior NRC Managers under the transitional criteria used prior to ROP implementation. The NRC no longer uses the "agency focus" designation for plants needing additional NRC oversight. Under the ROP, a licensee's performance is assessed based on their performance indicators (PIs) and NRC inspection findings. Based on a combination of these measures a licensee's performance is correlated to a column in the NRC Action Matrix, which details the type of NRC response and communication. Presently, Indian Point 2 plant performance is in the "Multiple/Repetitive Degraded Cornerstone" column of the NRC Action Matrix.

The quarterly assessment letters will inform the licensee and the public as to the performance of the plant over the most recent quarter (i.e. location in NRC's Action Matrix). The quarterly assessment letters can be found on the NRC web site at:  
<http://www.nrc.gov/NRR/OVERSIGHT/ASSESS/index.html>

In addition, copies of the quarterly assessment letters are available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of the NRC's document system (ADAMS). ADAMS is accessible from the NRC web site at: <http://www.nrc.gov/NRC/ADAMS/index.html>

- Q** Does the NRC review how unused money in the decommissioning fund are disposition by the licensee?
- A.** The NRC does not review how unused money in the decommissioning fund is disposition by the licensee. The NRC's regulatory responsibility is to ensure that adequate funds are present to ensure a safe and timely decommissioning of a site by the licensee.

Attachment: November 16, 2000 Public Meeting Presentation Material

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