



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 28, 1993

Docket No. 50-286

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M86162)

The Commission has issued the enclosed Amendment No. 135 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated April 9, 1993.

The amendment revises Technical Specifications Table 4.1-1 to change the frequency of primary auxiliary building (PAB) area temperature sensor testing to accommodate operating on a 24-month fuel cycle. This change followed the guidance provided in Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle," as applicable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "N. F. Conicella".

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 135 to DPR-64
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. Ralph E. Beedle
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Resident Inspector
Indian Point 3 Nuclear Power Plant
U.S. Nuclear Regulatory Commission
Post Office Box 337
Buchanan, New York 10511

Mr. Gerald C. Goldstein
Assistant General Counsel
Power Authority of the State
of New York
1633 Broadway
New York, New York 10019

Mr. Charles W. Jackson
Manager, Nuclear Safety and
Licensing
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York 10511

Mr. Robert G. Schoenberger, Acting
President
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Mr. John H. Garrity
Resident Manager
Indian Point 3 Nuclear Power Plant
Post Office Box 215
Buchanan, New York 10511

Mr. Peter Kokolakis
Director Nuclear Licensing - PWR
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

DATED: January 28, 1993

AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT UNIT 3

Docket File

NRC & Local PDRs

PDI-1 Reading

S. Varga, 14/E/4

J. Calvo, 14/A/4

R. Capra

C. Vogan

N. Conicella

OGC

D. Hagan, 3302 MNBB

G. Hill (2), P1-22

C. Grimes, 11/F/23

ACRS (10)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

J. Wermiel, 8/H/1

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 135
License No. DPR-64

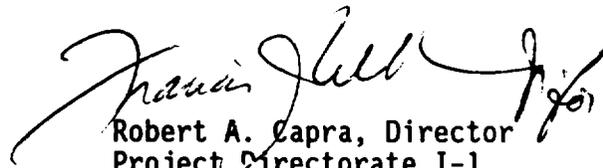
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 9, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 135, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 28, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 135

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Page

Table 4.1-1 (Sheet 3 of 5)

Insert Page

Table 4.1-1 (Sheet 3 of 5)

TABLE 4.1-1 (Sheet 3 of 5)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
23. Temperature Sensor in Auxiliary Boiler Feedwater Pump Building	N.A.	N.A.	18M	
24. Temperature Sensors in Primary Auxiliary Building				
a. Piping Penetration Area	N.A.	N.A.	24M	
b. Mini-Containment Area	N.A.	N.A.	24M	
c. Steam Generator Blowdown Heat Exchanger Room	N.A.	N.A.	24M	
25. Level Sensors in Turbine Building	N.A.	N.A.	18M	
26. Volume Control Tank Level	N.A.	18M	N.A.	
27. Boric Acid Makeup Flow Channel	N.A.	18M	N.A.	
28. Auxiliary Feedwater:				
a. Steam Generator Level	S	18M	Q	Low-Low
b. Undervoltage	N.A.	18M	18M	
c. Main Feedwater Pump Trip	N.A.	N.A.	18M	
29. Reactor Coolant System Subcooling Margin Monitor	D	18M	N.A.	
30. PORV Position Indicator	N.A.	N.A.	24M	Limit Switch
31. PORV Position Indicator	D	24M	24M	Acoustic Monitor
32. Safety Valve Position Indicator	D	24M	24M	Acoustic Monitor
33. Auxiliary Feedwater Flow Rate	N.A.	18M	N.A.	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated April 9, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3), Technical Specifications (TS). The requested changes would revise TS Table 4.1-1 to change the frequency of primary auxiliary building (PAB) area temperature sensor testing to accommodate operating on a 24-month fuel cycle. The licensee commenced operating on a 24-month fuel cycle, instead of the previous 18-month fuel cycle, with fuel cycle 9. Fuel cycle 9 started in August 1992. The proposed change follows the guidance provided in Generic Letter (GL) 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle," as applicable.

2.0 EVALUATION

The licensee considered the following factors in evaluating the PAB area temperature sensor testing interval extension from 18 to 24 months:

- Does on-line testing adequately demonstrate operability or are failures only being detected during these refueling tests?
- Did past equipment performance have an effect on system safety functions?
- Does performing the surveillance test at power present an unacceptable burden?

Each of the four steam generators (S/Gs) is provided with a drain connection and two blowdown connections that join to form a blowdown header. The four blowdown headers are routed from each respective S/G inside containment to the S/G blowdown recovery system which is located in the PAB. Each blowdown header contains two in-series containment isolation valves located outside of the containment in the pipe penetration area.

Redundant area temperature sensors are provided at three locations in the PAB in the vicinity of the S/G blowdown recovery piping for detection and mitigation of a high energy line break. Specifically, two resistance temperature detectors (RTDs) are located in each of the following areas:

- 1) piping penetration area at the 55' elevation
- 2) piping tunnel (mini-containment) area at the 35' elevation
- 3) heat exchanger room at the 18' elevation

The PAB area temperature sensor circuitry will automatically close the S/G blowdown header isolation valves (a total of 8 valves) if a high temperature is detected in any of these areas. In addition, high temperature in any of these areas will initiate an alarm in the control room.

The purpose of the PAB area temperature sensor refueling test is to demonstrate operability of the temperature sensors and associated circuitry. The licensee stated that the temperature sensors are considered operable if the associated S/G blowdown containment isolation valves close upon detection of a simulated high temperature condition in each of the three RTD locations in the PAB and if the associated alarm annunciates in the control room.

The current PAB area temperature sensors were installed in 1987. The licensee reviewed data from 1987 to 1992 related to PAB area temperature sensor testing and the data indicated that all tests were satisfactory. An evaluation of the calibration data indicated that the "as-found" values (at the actuation point) were within the calibration tolerances in all cases. Work control and significant occurrence report records also confirmed sensor reliability. Based on these results, and the fact that RTD drift is considered negligible, the licensee concluded that the PAB area temperature sensors testing interval could be extended. In addition, there was no evidence that temperature sensor performance was a function of the surveillance interval.

The S/G blowdown isolation valves are stroke-tested quarterly in accordance with the inservice testing (IST) program as required by Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code. This testing cycles each valve open and closed and verifies proper closing stroke time. The licensee stated that the valve IST also checks for unusual valve noises, checks proper limit switch operation, inspects the pneumatic valve operator tubing, and checks for proper remote and local valve indication. The licensee concluded that this on-line testing of the containment isolation valves would detect problems with the valves and related equipment.

The licensee also stated that although the PAB area temperature sensors could be tested at power, past test results, on-line testing, and the fact that there are redundant temperature sensors provide adequate assurance that the surveillance interval can be extended. Extending the surveillance interval will decrease the overall burden of surveillance testing. The licensee has evaluated the effect of the increase in the surveillance interval on safety for the proposed change and has concluded that the effect is small. The licensee has confirmed that historical plant maintenance and surveillance data

do not invalidate this conclusion. In addition, the increase in the surveillance interval to accommodate a 24-month fuel cycle does not invalidate any assumption in the IP3 licensing basis.

The staff has reviewed the information presented by the licensee and concludes that the proposed change does not have a significant effect on safety. Therefore, the proposed change is acceptable. In addition, the proposed change follows the guidance of GL 91-04, as applicable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 30199). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Nicola F. Conicella

Date: July 28, 1993

July 28, 1993

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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Sincerely,

Original signed by:
Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

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cc w/enclosures:

See next page

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NAME	CVogan <i>lv</i>	NConicella:avl	JWermiel <i>lv</i>	R Bachmann	RACapra <i>lv</i>
DATE	7/1/93	7.17/93	7/18/93	7/19/93	7/28/93

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