



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 5, 1994

Mr. William J. Cahill, Jr.
Executive Vice President - Nuclear
Generation
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M89994)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 152 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated July 25, 1994.

The amendment revises Table 3.6-1 (Non-Automatic Containment Isolation Valves Open Continuously or Intermittently for Plant Operation) and Table 4.4-1 (Containment Isolation Valves) to delete valves SI-1833A and B and add valves SI-MOV-1835A and B. The valves being deleted no longer perform a containment isolation function as a result of a modification which removed the boron injection tank. The valves being added are needed for testing the safety injection pumps.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 152 to DPR-64
2. Safety Evaluation

cc w/encls: See next page

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William J. Cahill, Jr.
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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DATED: October 5, 1994

AMENDMENT NO. 152 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT UNIT 3

Docket File

PUBLIC

PDI-1 Reading

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J. Zwolinski, 14/A/4

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C. Vogan

N. Conicella

OGC

D. Hagan, 3302 MNBB

G. Hill (2), P1-22

C. Grimes, 11/F/23

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ACRS (10)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated July 25, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 152, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 5, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 152

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

TABLE 3.6-1

TABLE 4.4-1 (page 2 of 7)

Insert Pages

TABLE 3.6-1

TABLE 4.4-1 (page 2 of 7)

TABLE 3.6-1

**NON-AUTOMATIC CONTAINMENT ISOLATION VALVES
OPEN CONTINUOUSLY OR INTERMITTENTLY FOR PLANT OPERATION**

VALVE NO.	VALVE NO.	VALVE NO.
AC-MOV-744	SI-MOV-850C	SWN-51-4
AC-MOV-1870	SI-MOV-1835A	SWN-44-5
AC-MOV-743	SI-MOV-1835B	SWN-51-5
SP-990C	SI-859A	SWN-71-1
AC-732	SI-859C	SWN-71-2
SI-MOV-885A	AC-752F	SWN-71-3
SI-MOV-885B	AC-753F	SWN-71-4
SI-MOV-888A	AC-752J	SWN-71-5
SI-MOV-888B	AC-753J	SA-24-1
CH-MOV-205	SWN-41-1	SA-24-2
CH-MOV-226	SWN-43-1	PS-PCV-1111-1
CH-227	SWN-41-2	PS-PCV-1111-2
CH-MOV-250A	SWN-43-2	SP-MOV-990A
CH-MOV-441	SWN-41-3	SP-MOV-990B
CH-MOV-250B	SWN-43-3	SI-1814A
CH-MOV-442	SWN-41-4	SI-1814B
CH-MOV-250C	SWN-43-4	SI-1814C
CH-MOV-443	SWN-41-5	HR-MOV-1882A
CH-MOV-250D	SWN-43-5	HR-MOV-1875A
CH-MOV-444	SWN-44-1	HR-MOV-1875B
SI-869A	SWN-51-1	HR-MOV-1876A
SI-869B	SWN-44-2	HR-MOV-1876B
SI-878A	SWN-51-2	PS-7
SI-878B	SWN-44-3	PS-8
SI-MOV-851A	SWN-51-3	PS-9
SI-MOV-850A	SWN-44-4	PS-10

TABLE 4.4-1 (Page 2 of 7)

CONTAINMENT ISOLATION VALVES			
<u>Valve No.</u>	<u>Penetration Number</u> ⁽¹⁾	<u>Test Fluid</u> ⁽²⁾	<u>Minimum Test Pressure (PSIG)</u> ⁽⁸⁾
CH-MOV-443	10	Water ⁽⁴⁾	47
CH-MOV-250D	10	Water ⁽⁴⁾	47
CH-MOV-444	10	Water ⁽⁴⁾	47
CH-MOV-222	11	Water ⁽⁴⁾	47
SP-AOV-956E	12	Water ⁽⁴⁾	47
SP-AOV-956F	12	Water ⁽⁴⁾	47
SI-869A	14	Water ⁽⁴⁾	47
SI-867A	14	Gas	43
SI-878A	14	Gas	43
SI-869B	14	Water ⁽⁴⁾	47
SI-867B	14	Gas	43
SI-878B	14	Gas	43
SI-MOV-1835A	15	Nitrogen ⁽⁴⁾	43
SI-MOV-1835B	15	Nitrogen ⁽⁴⁾	43
SI-MOV-851A	15	Water ⁽⁴⁾	47
SI-MOV-850A	15	Water ⁽⁴⁾	47
SI-MOV-850C	15	Water ⁽⁴⁾	47
SI-859A	16	Water ⁽⁴⁾	47
SI-859C	16	Water ⁽⁴⁾	47
NNE-1610	17	Gas	43
NNE-AOV-863	17	Gas	43
SP-AOV-956G	18	Water ⁽⁴⁾	47
SP-AOV-956H	18	Water ⁽⁴⁾	47
WD-AOV-1786	19	Water ⁽⁴⁾	47
WD-AOV-1787	19	Water ⁽⁴⁾	47



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 152 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated July 25, 1994, the Power Authority of the State of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3) Technical Specifications (TSs). The requested changes would revise Table 3.6-1 (Non-Automatic Containment Isolation Valves Open Continuously or Intermittently for Plant Operation) and Table 4.4-1 (Containment Isolation Valves) to delete valves SI-1833A and B and add valves SI-MOV-1835A and B. The valves being deleted no longer perform a containment isolation function as a result of a modification which removed the boron injection tank. The valves being added are needed for testing the safety injection pumps.

2.0 EVALUATION

Valves SI-1833A and B are located on a 3/4-inch line that bypasses the Boron Injection Tank (BIT) of the Safety Injection System (SIS). The original intent of the bypass line was to provide a means for performing periodic operability tests of SI pumps 32 and 33. During pump testing, the BIT was isolated and valves SI-1833A and B were opened. This type of test flow path was established to not disturb the concentrated boric acid in the BIT. The function of the BIT was to provide concentrated boric acid to the reactor coolant system to mitigate the consequences of postulated steam line break accidents. However, Amendment No. 139 eliminated the requirement for the BIT function because analysis confirmed that the concentrated boric acid could be eliminated without violating the design basis. The BIT still remains in place but the solution in the tank is now maintained at low boric acid concentration levels equivalent to that maintained in the Refueling Water Storage Tank (RWST). The BIT may now be included in the flow path during SI pump testing rendering the BIT bypass line unnecessary. Periodic testing of the SI pumps utilized a recirculation flow path that returned the fluid to the RWST. Following removal of the BIT bypass line, pump testing will be performed by directing the flow through the BIT, necessitating opening of one of the two BIT outlet valves, SI-MOV-1835A or SI-MOV-1835B. Accordingly, these valves need to be added to Table 3.6-1 to permit their opening when the reactor is not in the cold shutdown condition. Additionally, these valves will continue to perform the containment isolation function.

The proposed changes delete the BIT bypass line isolation valves and add the BIT outlet valves to the appropriate tables in the TSs. Removal of the bypass line isolation valves can now be accommodated since the SI pump flow test function of the valves is no longer needed as a result of the modification which removed the functional requirement for the BIT. Pump flow tests will be performed with the BIT included in the flow path, eliminating the need to open the subject valves on the BIT bypass line. Removal of the valves will also eliminate the potential for containment leakage due to degradation of the valves. Opening the BIT outlet valves assures the operability of the SIS which is designed to protect the reactor fuel from damage following a steam line break, and is consistent with the current provision that permits opening of the BIT bypass valves to effect testing. While the SI pump flow test is in progress, the SIS will be at a pressure well above the containment pressure, precluding the possibility of containment leakage.

Removal of valves SI-1833 A and B involves a modification to cut and cap the bypass line pipe to assure integrity of the containment and SIS. The new weld joints will be on the 3/4-inch capped line, and will form part of the high head SI piping boundary. The welds will be tested to verify integrity using an in-service pressure test and verifying zero leakage from the weld joints. Note that the function of the bypass line was to facilitate SI pump testing under the previous system configuration, and removal of this line does not alter the performance capabilities of the SIS.

In short, the modification to remove both isolation valves and capping the pipe assures the integrity of the containment and SIS. Additionally, removal of the isolation valves eliminates the potential for containment leakage which could have resulted from valve degradation. Finally, removal of the BIT bypass line and its associated isolation valves does not inhibit the ability to test the SI pumps.

Based on the information provided by the licensee, our review finds the proposed changes to the plant and TSs to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the

amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 42346). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Mario C. Gareri

Date: October 5, 1994

October 5, 1994

Mr. William J. Cahill, Jr.
Executive Vice President - Nuclear
Generation
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M89994)

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Sincerely,

Original signed by:
Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 152 to DPR-64
2. Safety Evaluation

cc w/encls: See next page

*See previous concurrence

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