

October 21, 1994

Mr. William J. Cahill, Jr.
Executive Vice President - Nuclear
Generation
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ENVIRONMENTAL ASSESSMENT REGARDING ONE-TIME SCHEDULAR EXEMPTION
FROM 10 CFR PART 50, APPENDIX J - INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M90471)

Dear Mr. Cahill:

By letter dated September 29, 1994, you requested an amendment to the Technical Specifications (TS) and exemptions to the *Code of Federal Regulation* requirements for the Indian Point Nuclear Generating Unit No. 3. The amendment and exemption would permit a one-time deferral of the Type C local leak rate tests (LLRTs) for Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. The leak rate test for these valves would be deferred until the next refueling outage (RFO 9/10), currently scheduled for the spring of 1996.

Enclosed for your information is a copy of the Environmental Assessment and Finding of No Significant Impact which supports your exemption request. Please note that the exemption request must be approved before the TS amendment request can be issued.

Sincerely,

Original signed by:

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: Environmental Assessment

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "N. F. Conicella".

Nicola F. Conicella, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: Environmental Assessment

cc w/encl: See next page

William J. Cahill, Jr.
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSION

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

ENVIRONMENTAL ASSESSMENT AND FINDING OF

NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.3, Type C Tests, to the Power Authority of the State of New York (the licensee) for the Indian Point Nuclear Generating Unit No. 3 (IP3), located in Westchester County, New York.

ENVIRONMENTAL ASSESSMENT

Identification Of Proposed Action:

The licensee would be exempt from the requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.3, to the extent that a one-time schedular extension would be allowed for performing Type C local leak rate tests (LLRTs) on Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. These LLRTs are currently required to be performed at intervals no greater than 30 months. The one-time schedular exemption would allow Type C LLRTs of the above listed valves to be deferred until the 9/10 refueling outage, which is currently scheduled for the spring of 1996.

By letter dated September 29, 1994, the licensee applied for a Technical Specifications (TSs) amendment and requested an Exemption from the Code of Federal Regulations requirements to allow Type C LLRTs to be deferred for the above listed RHR system valves until the 9/10 refueling outage.

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The Need for the Proposed Action:

The licensee commenced operating on 24-month fuel cycles, instead of the previous 18-month fuel cycles, starting with fuel cycle 9. Fuel cycle 9 started in August 1992. The requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.3, indicate that Type C LLRTs must be performed during each reactor shutdown for refueling at intervals no greater than 2 years (24 months). On January 12, 1993, the NRC staff issued an exemption to the licensee that allowed Type C LLRTs to be performed at intervals up to 30 months, thus, permitting operation on a 24-month fuel cycle.

Approximately 6 months after startup from the 8/9 refueling outage, IP3 began an extended unplanned non-refueling outage. Startup from the outage is currently expected for early 1995. After startup from the current outage, the plant will run until its next scheduled refueling outage (RFO 9/10) which is scheduled to begin in spring of 1996. In November and December of 1994 the RHR containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 are due for their Type C LLRTs. Currently, the interval for Type C testing of these valves is 30 months. These LLRTs are normally performed during a refueling outage when the reactor is defueled and the RHR system is not providing a source of cooling water. The current outage is a non-refueling outage, therefore, the reactor is not defueled and RHR system is providing core cooling water. The licensee's procedure and the system design require the RHR system to be out-of-service in order to perform the LLRTs. If the RHR system is taken out-of-service to perform the LLRTs the reactor would not have a reliable source of cooling water to remove decay heat. Therefore, the licensee is requesting a one-time schedular exemption to allow Type C

LLRTs of the above listed valves to be deferred until the 9/10 refueling outage, which is currently scheduled for the spring of 1996.

Environmental Impacts of the Proposed Action:

The proposed one-time exemption does not increase the probability or consequences of accidents previously analyzed and the proposed one-time exemption does not affect facility radiation levels or facility radiological effluents. The licensee has analyzed the results of previous LLRTs performed at IP3, and has provided the methodology used in extrapolating the previous LLRT data to the proposed one time increase in the surveillance interval. The licensee has provided a sound basis for concluding that the containment leakage rate would be maintained within acceptable limits with the one time extension of the LLRT interval to the refueling outage 9/10. Therefore, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed exemption.

With regard to potential nonradiological impacts, the proposed exemption only involves LLRTs on containment penetrations and isolation valves. They do not affect nonradiological plant effluents and have no other environmental impact. Therefore, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed exemption.

Alternatives to the Proposed Action:

Since the Commission has concluded that there are no significant environmental effects that would result from the proposed exemption, any alternatives with equal or greater environmental impacts need not be evaluated. The principal alternative would be to deny the licensee's request for exemption. Such action would not reduce the environmental impacts of plant operations.

Alternative Use of Resources:

This action does not involve the use of resources not previously considered in the "Final Environmental Statement Related to the Operation of Indian Point Nuclear Generating Plant Unit No. 3," dated February 1975.

Agencies and Persons Consulted:

The NRC staff consulted with the New York State official regarding the environmental impact of the proposed action. The State official had no comments.

FINDING OF NO SIGNIFICANT IMPACT

Based on the foregoing environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the exemption under consideration.

For further details with respect to this action, see the licensee's application for exemption dated September 29, 1994. This document is available for public inspection at the Commission's Public Document Room, 2120 L Street, NW., Washington, DC 20555 and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York 10610.

Dated at Rockville, Maryland, this 21st day of October 1994.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation