

August 30, 1989

Docket No. 50-286

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Mr. John C. Brons
 Executive Vice President, Nuclear Generation
 Power Authority of the State of New York
 123 Main Street
 White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: CORRECTED AMENDMENT NO. 87 (TAC NO. 73834)

On August 11, 1989 the Commission issued Amendment No. 87 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. Through an oversight, Technical Specification page 3.1-1 contained an error in that page 3.1-1 did not include the changes made to Technical Specification 3.A.1.b. that had been incorporated in License Amendment No. 84.

Enclosed is a copy of corrected page 3.1-1 that includes the changes incorporated by both Amendments No. 84 and 87. Please substitute this corrected Technical Specification page for the one that was issued with Amendment No. 87 on August 11, 1989.

We apologize for any inconvenience caused by this error.

Sincerely,

Original signed by

Joseph D. Neighbors, Senior Project Manager
 Project Directorate I-1
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc: See next page

[CORRECTED AMEND 87 73834]

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OFC : PDI-1	: PDI-1	: PDI-1	:	:	:	:	:
NAME : CVogan	: JNeighbors/bah	: RCapra	:	:	:	:	:
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Mr. John C. Brons
Power Authority of the State
of New York

Indian Point Nuclear Generating
Unit No. 3

cc:

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Resident Inspector
Indian Point Nuclear Generating
U.S. Nuclear Regulatory Commission
Post Office Box 337
Buchanan, New York 10511

Mr. Gerald C. Goldstein
Assistant General Counsel
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Mr. Robert L. Spring
Nuclear Licensing Engineer
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Mr. Phillip Bayne, President
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Mr. A. Klausmann, Vice President
Quality Assurance
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Mr. William Josiger
Resident Manager
Indian Point 3 Nuclear Power Plant
Post Office Box 215
Buchanan, New York 10511

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Mr. George M. Wilverding, Manager
Nuclear Safety Evaluation
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Mr. F. X. Pindar
Quality Assurance Superintendent
Indian Point 3 Nuclear Power Plant
Post Office Box 215
Buchanan, New York 10511

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. R. Beedle, Vice President
Nuclear Support
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

cc

Mr. Peter Kokolakis, Director
Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

Mr. S. S. Zulla, Vice President
Nuclear Engineering
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Vice President
Nuclear Operations
Power Authority of the State
of New York
123 Main Street
White Plains, New York 10601

Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

3. LIMITING CONDITIONS FOR OPERATION

For the case where no exception time is specified for inoperable components, this time is assumed to be zero.

In the event that service water temperature exceeds 90°F, the unit shall be placed in at least hot shutdown within the next seven hours and be in at least cold shutdown within the following thirty hours unless service water temperature is reduced to 90°F or less within these time intervals as measured from initial discovery or until the reactor is placed in a condition where this service water temperature is not applicable.

3.1 REACTOR COOLANT SYSTEM

Applicability

Applies to the operating status of the Reactor Coolant System; operational components; heatup; cooldown; criticality; activity; chemistry and leakage.

Objective

To specify those limiting conditions for operation of the Reactor Coolant System which must be met to ensure safe reactor operation.

Specification

A. OPERATIONAL COMPONENTS

1. Coolant Pumps

- a. When a reduction is made in the boron concentration of the reactor coolant, at least one reactor coolant pump or one residual heat removal pump (connected to the Reactor Coolant System) shall be in operation.
- b. (1) When the reactor coolant system T_{avg} is greater than 350°F and electrical power is available to the reactor coolant pumps, and as permitted during special plant evolutions, at least one reactor coolant pump shall be in operation. All reactor coolant pumps may be de-energized for up to 1 hour provided no operations are permitted that would cause dilution of the reactor coolant system boron concentration, and core outlet temperature is maintained at least 10°F below saturation temperature.

(2) When the reactor is subcritical and reactor coolant system T_{avg} is greater than 350°F, control bank withdrawal shall be prohibited unless four reactor coolant pumps are operating.
- c. When the reactor coolant system T_{avg} is greater than 200°F and less than 350°F, and as permitted during special plant evolutions, at least one reactor coolant pump or one residual heat removal pump (connected to the Reactor Coolant System) shall

3.1-1

Amendment No. ~~48~~, ~~53~~, ~~82~~, ~~84~~

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