

June 11, 1986

Docket No. 50-247

Mr. John D. O'Toole  
Vice President  
Nuclear Engineering and Quality Assurance  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place  
New York, New York 10003

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N. Thompson	E. Butcher
B. Mann	C. Berlinger

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 113 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by telecopy dated May 23, 1986.

The amendment revises the Technical Specifications to permit on a one-time only basis relief from Technical Specification 3.4.B which currently requires that within 72 hours after discovery that a minimum of twenty ASME Code approved steam-relieving main steam valves are not operable (except for testing), the reactor shall be placed in the hot shutdown condition within the next 12 hours. The change extends the 72-hour limiting condition of operation for a period not to exceed an additional two weeks. In addition, the Specification limits power operation to less than or equal to 78%.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager  
PWR Project Directorate #3  
Division of PWR Licensing-A, NRR

Enclosures:

1. Amendment No. 113 to DPR-26
2. Safety Evaluation

cc: w/enclosures  
See next page  
\*SEE PREVIOUS PAGE FOR CONCURRENCE

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DATE :5/29/86	:5/ /86	:5/29/86	:6/3/86	:5/29/86	:

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NAME :CVogan	:MSlosson;ps	:CBerlinger	:SVarga	:	:
DATE :5/29/86	:5/29/86	:5/ /86	:5/ /86	:5/ /86	:

Mr. John D. O'Toole  
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of New York, Inc.

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 113  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated May 23, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

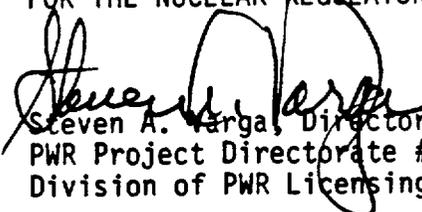
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 113, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective on May 23, 1986.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Yarga, Director  
PWR Project Directorate #3  
Division of PWR Licensing-A, NRR

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 11, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 113 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

3.4-1

Insert Page

3.4-1

### Applicability

Applies to the operating status of the Steam and Power Conversion System.

### Objective

To define conditions of the turbine cycle steam-relieving capacity. Auxiliary Feedwater System and City Water System operation is necessary to ensure the capability to remove decay heat from the core.

### Specification

- A. The reactor shall not be heated above 350°F unless the following conditions are met:
- (1) A minimum ASME code approved steam-relieving capability of twenty (20) main steam valves shall be operable (except for testing).
  - (2) Three auxiliary feedwater pumps each capable of pumping a minimum of 400 gpm must be operable.
  - (3) A minimum of 360,000 gallons of water in the condensate storage tank and a backup supply from the city water supply.
  - (4) Required system piping, valves, and instrumentation directly associated with the above components operable.
  - (5) The main steam stop valves are operable and capable of closing in five seconds or less.
  - (6) The total iodine activity of I-131 and I-133 on the secondary side of the steam generator shall be less than or equal to 0.15 uCi/cc.
- B. Except as modified by 3.4.C below, if any of the conditions of 3.4.A above cannot be met within 72 hours\*, the reactor shall be placed in the hot shutdown condition within the next 12 hours and subsequently cooled below 350°F using normal operating procedures.

\* On a one time basis, the 72 hour action statement for main steam safety valves MS-49A and MS-45C may be extended for an additional 2 week period ending 4:45 p.m. on June 7, 1986 provided the remaining 18 safety valves are operable and the high flux trip setpoint is reduced to 78% of rated thermal power.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 113 TO FACILITY OPERATING LICENSE NO. DPR-26  
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2  
DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated May 23, 1986 Consolidated Edison submitted an immediate Technical Specification change to the Indian Point Nuclear Generating Unit No. 2 (IP-2) license. This change would permit on a one-time basis relief from Technical Specification 3.4.B which currently requires that within 72 hours after discovery that a minimum of twenty (20) ASME Code approved steam-relieving main steam valves are not operable (except for testing), the reactor shall be placed in the hot shutdown condition within the next 12 hours. The change would extend the 72 hour limiting condition of operation for a period not to exceed an additional two weeks. In the May 23 submittal, the licensee indicated that on Wednesday, May 21, 1986 the above limiting condition was entered. The situation was caused by malfunctioning equipment which could not be foreseen or avoided. Efforts to perform corrective maintenance were unsuccessful over the next 2 days. Therefore it was determined that immediate license amendment action was warranted.

2.0 REVIEW AND EVALUATION

The IP-2 design provides 5 safety valves per steam generator, for a total of 20 valves. Presently two of these valves (one on SG #21 and the other on SG #23) are inoperable. The IP-2 TS 3.4.A(1) requires that a minimum of 20 ASME Code safety valves be operable when above 350°F. If this requirement cannot be met within 72 hours, TS 3.4.B requires the reactor to be placed in hot shutdown within the next 12 hours. The licensee has requested that, on a one-time basis, this period be extended to a maximum of 2 weeks at which time the inoperable valves would be repaired, provided the remaining 18 safety valves are operable and the high flux trip setpoint is reduced to less than or equal to 78% of rated thermal power.

The licensee has stated that 92% of the rated thermal power would ensure meeting the ASME Code requirements with regard to available relieving capacity for the steam generator with the lowest relieving capacity (SG #21).

The Standard Technical Specification (STS) for Westinghouse PWRs, NUREG-0452, Rev. 4, permits operation with one or more inoperable safety valves if the power range high flux setpoint is reduced to 87% of rated

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thermal power with one inoperable safety valve on any one steam generator. The ratio of the SG #21 maximum relieving capacity to the maximum relieving capacity of an intact subsystem (i.e., a subsystem having all safety valves operable) is 0.78. Therefore, 78% has been conservatively chosen as the maximum rated thermal power for interim operation with less than 20 intact steam generator safety valves. The licensee states, and the staff concurs that operation at less than or equal to 78% of rated thermal power does not invalidate the design basis loss of load transient.

### 3.0 EVALUATION OF NO SIGNIFICANT HAZARDS DETERMINATION

The Commission has provided criteria for determining whether an amendment to an operating license involves no significant hazards consideration in 10 CFR 50.92 (48 FR 14871). An amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with the amendment will not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; (2) create the possibility of a new or different kind of accident previously evaluated; or (3) involve a significant reduction in margin of safety.

The change to the limiting condition of operation for the steam generator safety valves will not increase the probability or the consequences of an accident previously evaluated because the limit on power to 78% of rated thermal power will reduce the consequences of a full loss of load accident and ensure meeting the required heat removal capability at the reduced power level. The change will not create the possibility of a new or different kind of accident from any previously evaluated since the change only involves a reduction in power in order to compensate for any loss of steam relieving capability of MS-49A and MS-45C. The change does not involve a significant reduction in margin of safety because the power reduction compensates for any reduction in margin caused by inoperability of MS-49A and MS-45C. The margin associated with all of the safety valves operable at 100% is not reduced at all when the power range high flux setpoint is reduced to 78% with the two valves, MS-49A and MS-45C inoperable.

### 4.0 FINDING ON EXISTENCE OF EMERGENCY SITUATION

In its submittal of May 23, 1986 Consolidated Edison explains the circumstances that led to the situation where, if the existing limiting conditions for operation in the Indian Point 2 Technical Specifications concerning the operability of the main steam safety valves were fully implemented, a forced shutdown of Indian Point 2 would result. The staff has reviewed the submittal and concluded that the situation could not reasonably be anticipated by Consolidated Edison, the failure to act in a timely way would result in shutdown of Indian Point 2, and therefore an emergency situation existed. For that reason the Commission issued the amendments under the provision of 10 CFR 50.91(a)(5).

## 5.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

## 6.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

## 7.0 FINDING

We have concluded, based on the consideration discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 11, 1986

### PRINCIPAL CONTRIBUTOR:

B. Mann