

July 22, 1986

Docket No. 50-247

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Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No.114 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 18, 1985.

The amendment revises the Technical Specifications to provide for reporting of relief and safety valve challenges in the monthly operating report and to conform the wording concerning the monthly operating report to the Standard Technical Specification wording. The June 18, 1985 amendment application concerned several topics. The other topics are the subject of separate amendments.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

MS

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PDR ADDCK 05000247
P PDR

Marylee M. Slosson, Project Manager
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Enclosures:

1. Amendment No.114 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

per memo 7/7

MS
7/18/86

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MSlosson
8/2/86

PD#3
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6/ /86

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6/27/86

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 18, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

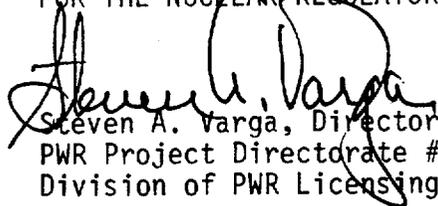
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Director
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance:
July 22, 1986



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

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Monthly Operating Report

- 6.9.1.7 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORV's or Pressurizer safety valves shall be submitted on a monthly basis to the Director, Office of Management and Program Analysis, U.S. Nuclear Regulatory Commission, Washington, D C. 20555, no later than the 15th of each month following the calendar month covered by the report.

Special Reports

- 6.9.2 Special reports shall be submitted to the NRC Regional Administrator of the Region I Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:
- a. Each containment integrated leak rate test shall be the subject of a summary technical report including results of the local leak rate test since the last report. The report shall include analyses and interpretations of the results which demonstrate compliance in meeting the leak rate limits specified in the Technical Specifications.
 - b. Inoperable fire protection and detection equipment (Specification 3.13).
 - c. Sealed source leakage in excess of limits (Specification 4.15).
 - d. The complete results of the steam generator tube inservice inspection (Specification 4.13.C).
 - e. Radioactive effluents (Specification 3.9).
 - f. Radiological environment monitoring (Specification 4.11)
 - g. Meteorological monitoring instrumentation (Specification 3.15).
 - h. Inoperable radiation and hydrogen monitoring instrumentation (Specification 3.5) outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.
 - i. Operation of overpressure protection system (Specification 3.1.A.4)

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. Reportable Event Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material or record.

6.10.2 The following record shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material releases to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

Record Retention (continued)

- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual except as noted in 6.10.1.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SNSC and the NFSC.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Record of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- n. *Records of the service lives of all snubbers listed in Table 3.12-1 including the date at which the service life commences and associated installation and maintenance records.

6.11 Radiation Protection Program

Procedure for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 High Radiation Area

6.12.1 As an acceptable alternate to the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry to such areas and the keys shall be maintained under the administrative control of the Watch Supervisor on duty.

* The documentation referred to herein is required for all snubbers beginning with those replaced following the issuance of the amendment.

6.13 Environmental Qualification

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of License No. DPR-26 dated October 24, 1980.

6.13.2. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines of NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.14 Process Control Program (PCP)

6.14.1 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 - b. A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the (SNSC).
2. Shall become effective upon review and acceptance by the (SNSC).

6.15 Offsite Dose Calculation Manual (ODCM)

6.15.1 The ODCM shall be approved by the Commission prior to implementation.

6.15.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluation justifying the change(s);
 - b. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 - c. Documentation of the fact the change has been revised and found acceptable by the (SNSC).
2. Shall become effective upon review and acceptance by the (SNSC).

6.16 Major Changes to Radioactive Liquid, Gaseous and Solid Waste

6.16.1 Licensee initiated major changes to the radioactive waste systems (liquid, gaseous and solid shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change was made. The discussion of each change shall contain:

- a. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR Part 50.59.
- b. Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;
- c. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;
- d. An evaluation of the change, which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments thereto;

- e. An evaluation of the change, which shows the expected maximum exposures to individual in the Unrestricted Area and to the general population that differ from those previously estimated in the license application and amendments thereto; *
- f. A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period to when the changes are to be made;
- g. An estimate of the exposure to plant operating personnel as a result of the change; and
- h. Documentation of the fact that the change was reviewed and found acceptable by the (SNSC).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letter dated June 18, 1985, Consolidated Edison requested several changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications. One of the issues contained in the amendment application concerned the monthly operating report. Specifically, Consolidated Edison requested that the wording describing the monthly operating report be revised to be consistent with the wording in the Westinghouse Standard Technical Specifications. This request involves removal of information of a descriptive nature concerning the content of the monthly operating report. It also involves adding a requirement to report relief valve and safety valve challenges in the monthly operating report. The other issues contained in the June 18, 1985 amendment application are the subject of separate safety evaluations.

EVALUATIONS

By letter dated September 19, 1977, Consolidated Edison was requested to modify the content of the required Monthly Operating Report. The Technical Specifications at that time referenced Regulatory Guide 1.16 concerning the content of the report. Consequently by Amendment 37, the Technical Specifications were revised to include a description of the content of the Monthly Operating Report as included in the September 19, 1977 letter. The June 18, 1985 amendment request concerning the Monthly Operating Report revises the Technical Specification Section 6.9.1.7 to eliminate the detailed description of the Monthly Operating Report. The wording submitted by Consolidated Edison is in accordance with that contained in the Westinghouse Standard Technical Specifications.

In order to ensure Consolidated Edison's continued use of the format requested in the September 19, 1977 letter, by letter dated March 17, 1986, Consolidated Edison was requested to commit to the use of that format. By letter dated June 9, 1986, Consolidated Edison provided such a commitment. On this basis we have determined that the removal of the detailed description of the content of the Monthly Operating Report contained in the Technical Specifications is acceptable. It should be noted that the use of the forms contained in the September 19, 1977 letter is not safety significant but is that desired by the NRC staff.

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Also included in the June 18, 1985 amendment application is the request to include in Section 6.9.1.7 the requirement to report relief and safety valve challenges in the Monthly Operating Report. Generic Letter 82-16 dated September 20, 1982 and NRC letter to Consolidated Edison dated March 20, 1985 requested that this requirement be included in the Technical Specifications. Therefore, the requested revision is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 22, 1986

PRINCIPAL CONTRIBUTOR

M. Slosson