

November 8, 1990

Docket No. 50-286

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Mr. John C. Brons
 Executive Vice President - Nuclear Generation
 Power Authority of the State of New York
 123 Main Street
 White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT 3 (TAC NO. 77625)

The Commission has issued the enclosed Amendment No. 105 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 19, 1990.

The amendment revises the Technical Specifications to remove Containment Isolation Valves UH-37 and UH-38 from Tables 3.6-1 and Table 4.4-1 (page 5 of 7).

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by
Daniele Oudinot for

Joseph D. Neighbors, Senior Project Manager
 Project Directorate I-1
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.105 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1:LA
 CVogan
 10/4/90

PDI-1:PM
 JNeighbors:avl
 10/11/90

P.R.
 SPLB
 McCracken
 10/12/90

OGC *elb*
 10/24/90

RAC
 D:PDI-1
 RACapra
 11/07/90

DOCUMENT NAME: AMEND TAC NO. 77625

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

November 8, 1990

Docket No. 50-286

Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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Sincerely,

A handwritten signature in cursive script, appearing to read "Joseph D. Neighbors".

Joseph D. Neighbors, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.105 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

Mr. John C. Brons
Power Authority of the State
of New York

Indian Point 3 Nuclear Power Plant

cc:

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New York, New York 10271



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated September 19, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 105, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 8, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 105

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

Table 3.6-1
Table 4.4-1 (page 5 of 7)

Insert Pages

Table 3.6-1
Table 4.4-1 (page 5 of 7)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

INTRODUCTION

By letter dated September 19, 1990, the Power Authority of the State of New York (the licensee) requested an amendment to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment would revise the Technical Specifications to delete Containment Isolation Valves UH-37 and UH-38.

DISCUSSION & EVALUATION

Containment Isolation Valves UH-37 and UH-38 are on the Auxiliary Steam Supply and Condensate Return (ASC) lines, respectively. These lines originally were designed to supply steam to the Containment Unit Heaters. Since the temperature in containment is maintained without the use of the Unit Heaters, auxiliary steam is no longer required to be supplied to containment. Since Valves UH-37 and UH-38 are containment isolation valves, they are required per the Technical Specifications to be maintained and tested.

The licensee proposes to remove Valves UH-37 and UH-38 and to cut and cap the adjacent piping near both sides of their respective containment penetrations. This modification would enhance the integrity of the containment penetrations by assuring that the capped piping sections through the penetrations are leak tight.

The proposed Technical Specification would delete Valves UH-37 and UH-38 from Tables 3.6-1 and 4.4-1 to reflect the changes associated with the plant modification described above. The changes will eliminate the need for maintaining and testing Valves UH-37 and UH-38.

We have reviewed this change and conclude that it would enhance containment integrity and is, therefore, acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 8, 1990

PRINCIPAL CONTRIBUTOR:

J. D. Neighbors

TABLE 4.4-1 (Page 5 of 7)

CONTAINMENT ISOLATION VALVES			
<u>Valve No.</u>	<u>Penetration Number</u> ⁽¹⁾	<u>Test Fluid</u> ⁽²⁾	<u>Minimum Test Pressure (PSIG)</u> ⁽⁸⁾
SWN-44-5	40	Water ⁽⁶⁾	47
SWN-51-5	40	Water ⁽⁶⁾	47
SWN-71-1	40	Water ⁽⁶⁾	47
SWN-71-2	40	Water ⁽⁶⁾	47
SWN-71-3	40	Water ⁽⁶⁾	47
SWN-71-4	40	Water ⁽⁶⁾	47
SWN-71-5	40	Water ⁽⁶⁾	47
SA-24-1	41	Water ⁽⁴⁾	47
SA-24-2	41	Water ⁽⁴⁾	47
VS-FCV-1170	48	Gas ⁽⁷⁾	43
VS-FCV-1171	48	Gas ⁽⁷⁾	43
VS-FCV-1172	49	Gas ⁽⁷⁾	43
VS-FCV-1173	49	Gas ⁽⁷⁾	43
VS-FCV-1190	50	Gas ⁽⁷⁾	43
VS-FCV-1191	50	Gas ⁽⁷⁾	43
VS-FCV-1192	50	Gas ⁽⁷⁾	43
SP-MOV-990A	51	Nitrogen ⁽⁴⁾	43
SP-MOV-990B	51	Nitrogen ⁽⁴⁾	43
SP-AOV-956A	52	Water ⁽⁴⁾	47
SP-AOV-956B	52	Water ⁽⁴⁾	47
SP-AOV-956C	53	Water ⁽⁴⁾	47
SP-AOV-956D	53	Water ⁽⁴⁾	47
SI-1814A	54	Gas	43
SI-1814B	55	Gas	43
SI-1814C	56	Gas	43
SP-SOV-506	57	Gas	43
SP-SOV-507	57	Gas	43

TABLE 3.6-1

NON-AUTOMATIC CONTAINMENT ISOLATION VALVES
OPEN CONTINUOUSLY OR INTERMITTENTLY FOR PLANT OPERATION

VALVE NO.	VALVE NO.	VALVE NO.
AC-MOV-744	SI-MOV-850C	SWN-51-4
AC-MOV-1870	SI-1833A	SWN-44-5
AC-MOV-743	SI-1833B	SWN-51-5
SP-990C	SI-859A	SWN-71-1
AC-732	SI-859C	SWN-71-2
SI-MOV-885A	AC-752F	SWN-71-3
SI-MOV-885B	AC-753F	SWN-71-4
SI-MOV-888A	AC-752J	SWN-71-5
SI-MOV-888B	AC-753J	SA-24-1
CH-MOV-205	SWN-41-1	SA-24-2
CH-MOV-226	SWN-43-1	PS-PCV-1111-1
CH-227	SWN-41-2	PS-PCV-1111-2
CH-MOV-250A	SWN-43-2	
CH-MOV-441	SWN-41-3	SP-MOV-990A
CH-MOV-250B	SWN-43-3	SP-MOV-990B
CH-MOV-442	SWN-41-4	SI-1814A
CH-MOV-250C	SWN-43-4	SI-1814B
CH-MOV-443	SWN-41-5	SI-1814C
CH-MOV-250D	SWN-43-5	HR-MOV-1882A
CH-MOV-444	SWN-44-1	HR-MOV-1875A
SI-869A	SWN-51-1	HR-MOV-1875B
SI-869B	SWN-44-2	HR-MOV-1876A
SI-878A	SWN-51-2	HR-MOV-1876B
SI-878B	SWN-44-3	PS-7
SI-MOV-851A	SWN-51-3	PS-8
SI-MOV-850A	SWN-44-4	PS-9
		PS-10