



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 15, 2000

Mr. Joel Sorensen
Site General Manager
Prairie Island Nuclear Generating Plant
Nuclear Management Company, LLC
1717 Wakonade Drive East
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: MODIFICATIONS TO MOTOR CONTROL
CENTERS (TAC NOS. MA9051 AND MA9052)

Dear Mr. Sorensen:

The Commission has issued the enclosed Amendment No. 155 to Facility Operating License No. DPR-42 and Amendment No. 146 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to the Northern States Power Company's (NSP's) application dated May 15, 2000.

NSP was subsequently succeeded by Nuclear Management Company, LLC (NMC), as the licensed operator of Prairie Island, Units 1 and 2. By letter dated October 5, 2000, NMC requested the staff continue to process and disposition licensing actions previously docketed and requested by NSP.

The amendments revise TS 3.7.B.6 to explicitly allow de-energizing motor control center (MCC) 1T1 or MCC 1T2 for up to 72 hours to accommodate installation of transfer switches for the MCCs.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Tae Kim, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures: 1. Amendment No. 155 to DPR-42
2. Amendment No. 146 to DPR-60
3. Safety Evaluation

cc w/encls: See next page

NRR-058

December 15, 2000

Mr. Joel Sorensen
Site General Manager
Prairie Island Nuclear Generating Plant
Nuclear Management Company, LLC
1717 Wakonade Drive East
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/RA/
Tae Kim, Senior Project Manager, Section 1
Project Directorate III
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Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

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cc w/encl: See next page

DISTRIBUTION

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PDIII-1 Reading ACRS
CCraig WBeckner
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RBouling RLanksbury, RGN-III

*No significant changes to SE
**Previously Concurred

OFFICE	PDIII-1/PM	PDIII-1/LA	EEIB/SC*	OGC**	PDIII-1/SC
NAME	TKim <i>TJK</i>	RBouling <i>RBouling</i>	CHolden	AHodgedon	CCraig <i>CC</i>
DATE	12/15/00	12/15/00	08/30/00	12/12/00	12/15/00

Prairie Island Nuclear Generating Plant,
Units 1 and 2

cc:

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October 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. DPR-42

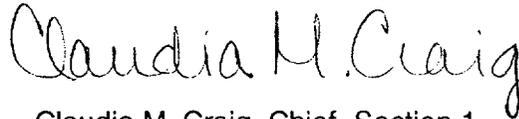
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 15, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 155 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 15, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 155

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

TS.3.7-3

INSERT

TS.3.7-3

- 3.7.B.5. D1 and D2 (Unit 2: D5 and D6) diesel generators may be inoperable for 2 hours provided the two required paths from the grid to the unit 4 kV safeguards distribution system are OPERABLE and the OPERABILITY of the two required paths from the grid are verified OPERABLE within 1 hour.
6. Except as specified in (a) and (b) below, one 4 kV safeguards bus (and/or its associated 480 V buses including associated safeguards motor control centers) may be inoperable or not fully energized for 8 hours provided the redundant 4 kV safeguards bus and its associated 480 V safeguards buses are verified OPERABLE and the diesel generator and safeguards equipment associated with the redundant train are OPERABLE.
- a. Motor control center 1T1 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T2 and its associated 480 V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T2 are OPERABLE.
- b. Motor control center 1T2 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T1 and its associated 480 V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T1 are OPERABLE.
7. One battery charger may be inoperable for 8 hours provided, (a) its associated battery is OPERABLE, (b) its redundant counterpart is verified OPERABLE, and (c) the diesel generator and safeguards equipment associated with its counterpart are OPERABLE.
8. One battery may be inoperable for 8 hours provided that the other battery and both battery chargers remain OPERABLE.
9. In addition to the requirements of Specification TS.3.7.A.7 a second inverter supplying Instrument AC Panels 111, 112, 113, and 114 may (Unit 2 panels 211, 212, 213 and 214) be powered from an inverter bypass source for 8 hours.

Prairie Island Unit No. 1
Prairie Island Unit No. 2

Amendment No. 91, 103, 110, 155
Amendment No. 84, 98, 103, 146



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.146
License No. DPR-60

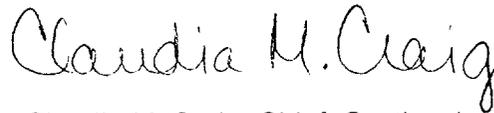
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 15, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 146 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 15, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 146

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

TS.3.7-3

TS.3.7-3

- 3.7.B.5. D1 and D2 (Unit 2: D5 and D6) diesel generators may be inoperable for 2 hours provided the two required paths from the grid to the unit 4 kV safeguards distribution system are OPERABLE and the OPERABILITY of the two required paths from the grid are verified OPERABLE within 1 hour.
6. Except as specified in (a) and (b) below, one 4 kV safeguards bus (and/or its associated 480 V buses including associated safeguards motor control centers) may be inoperable or not fully energized for 8 hours provided the redundant 4 kV safeguards bus and its associated 480 V safeguards buses are verified OPERABLE and the diesel generator and safeguards equipment associated with the redundant train are OPERABLE.
- a. Motor control center 1T1 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T2 and its associated 480 V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T2 are OPERABLE.
- b. Motor control center 1T2 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T1 and its associated 480 V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T1 are OPERABLE.
7. One battery charger may be inoperable for 8 hours provided, (a) its associated battery is OPERABLE, (b) its redundant counterpart is verified OPERABLE, and (c) the diesel generator and safeguards equipment associated with its counterpart are OPERABLE.
8. One battery may be inoperable for 8 hours provided that the other battery and both battery chargers remain OPERABLE.
9. In addition to the requirements of Specification TS.3.7.A.7 a second inverter supplying Instrument AC Panels 111, 112, 113, and 114 may (Unit 2 panels 211, 212, 213 and 214) be powered from an inverter bypass source for 8 hours.

Prairie Island Unit No. 1
 Prairie Island Unit No. 2

Amendment No. 97, 103, 110, 155
 Amendment No. 84, 98, 103, 146



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. DPR-42

AND AMENDMENT NO. 146 TO FACILITY OPERATION LICENSE NO. DPR-60

NUCLEAR MANAGEMENT COMPANY, LLC

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By application dated May 15, 2000, the licensee requested changes to the Technical Specifications (TSs) for the Prairie Island Nuclear Generating Plant, Units 1 and 2. The proposed changes would modify TS 3.7.B.6 to explicitly allow de-energizing motor control center (MCC) 1T1 and MCC 1T2 for up to 72 hours. The licensee chose 72 hours as a limit based on the fact that 72 hours is the most limiting allowed outage time for any equipment powered by MCCs 1T1 and 1T2. Current TS 3.7.B.6 makes no explicit provision for individual MCCs being out of service. The licensee stated that its past practice has been to interpret TS 3.7.B.6 as providing an 8-hour allowed out of service time for individual MCCs.

MCC 1T1 and MCC 1T2 supply power to loads shared between the two units. Each MCC has two 480V sources, one from each unit. The electrical power to MCC 1T1 can be supplied from Unit 1 480V Bus 112 or Unit 2 480V Bus 212. Likewise, power to MCC 1T2 can be supplied from Unit 1 480V Bus 122 or Unit 2 480V Bus 212. However, the only isolation device for each MCC is a single breaker located at each 480V bus. There is no isolation or breakers at the MCC end of the 480V cables. Therefore, when MCC 1T1 or MCC 1T2 is energized, both cable feeds are energized to the source breakers regardless of which bus is supplying power to the MCC. The 480V source breakers for each MCC are electrically interlocked to prevent tying the opposite Unit 480V buses together via the MCC.

All power sources to the 480V bus are isolated during maintenance except the cable and the stabs of the source breaker in the de-energized 480V bus under maintenance. The energized cable and breaker stabs in the otherwise de-energized bus causes a safety hazard for personnel performing bus maintenance. The licensee stated that there have been two near-miss events, one in November of 1992 and one in April of 1999, involving energized MCC cable in an otherwise de-energized 480V bus. As a preventive measure, the licensee is planning to install a transfer switch for each MCC that would eliminate the potential personnel safety hazard by completely de-energizing the cables and the breakers stabs associated with the maintenance. The estimated time needed to install a transfer switch on each MCC is approximately 48 hours. Therefore, the proposed changes would explicitly allow (one time only for the expressed purpose of installing the transfer switches) the out of service time necessary to accommodate the installation of transfer switches for the MCCs without an unnecessary dual unit shutdown.

2.0 EVALUATION

The current TS 3.7.B.6 specifies the following:

One 4 kV safeguards bus (and/or its associated 480 V buses including associated safeguards motor control centers) may be inoperable or not fully energized for 8 hours provided the redundant 4 kV safeguards bus and its associated 480 V safeguards buses are verified OPERABLE and the diesel generator and safeguards equipment associated with the redundant train are OPERABLE.

The proposed amendments revise TS 3.7.B.6 as indicated below:

Except as specified in (a) and (b) below, one 4 kV safeguards bus (and/or its associated 480 V buses including associated safeguards motor control centers) may be inoperable or not fully energized for 8 hours provided the redundant 4 kV safeguards bus and its associated 480 V safeguards buses are verified OPERABLE and the diesel generator and safeguards equipment associated with the redundant train are OPERABLE.

- a. *Motor control center 1T1 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T2 and its associated 480V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T2 are OPERABLE.*
- b. *Motor control center 1T2 may be de-energized for up to 72 hours (for the purposes of installing transfer switches as part of Modification 99EB01, only) provided motor control center 1T1 and its associated 480V bus is verified OPERABLE and the diesel generator and safeguards equipment associated with 1T1 are OPERABLE.*

The amendments modify TS 3.7.B.6 by appending subparagraphs (a) and (b) to explicitly allow de-energizing motor control center (MCC) 1T1 and MCC 1T2 for up to 72 hours for the purpose of installing transfer switches per Modification 99EB01. The licensee stated that the estimated time required to install each transfer switch is about 48 hours. The current TS 3.7.B.6 makes no explicit provision for individual MCCs being out of service. The licensee stated that its past practice has been to interpret TS 3.7.B.6 as providing an 8-hour allowed out of service time for individual MCCs. The allowed outage time of 72 hours in the amendments was chosen by the licensee because (1) it will provide enough time for the installation of each transfer switch, and (2) it is less than the most restrictive allowed out of service time for the plant loads that are powered by either of the MCC.

MCC 1T1 and MCC 1T2 supply power to loads shared between the two units. Each MCC has two 480V sources, one from each unit. The electrical power to MCC 1T1 can be supplied from Unit 1 480V Bus 112 or Unit 2 480V Bus 212. Likewise, power to MCC 1T2 can be supplied from Unit 1 480V Bus 122 or Unit 2 480V Bus 212. The plant loads that are powered by MCC 1T1 and MCC 1T2 and their associated TS requirements are listed below:

- Control Room Ventilation - MCC 1T1 powers Train A and MCC 1T2 powers Train B. TS 3.13.A2 allows one train to be out of service for up to 7 days.
- Spent Fuel Pool Pumps - MCC 1T1 powers the Spent Fuel Pool Pump No. 121 and MCC 1T2 powers the Spent Fuel Pool Pump No. 122. There are no TS requirements associated with the function of these pumps.
- Miscellaneous Radiation Monitors - The requested change ensures that radiation monitors are available. Prairie Island TS requires that radiation monitors that initiate isolation of the containment purge system be verified operable prior to core alteration. The installation of the transfer switches will not be implemented if the containment purge is in service.
 - Radiation monitors R-30 and R-37 start the Auxiliary Building Special Ventilation. Prairie Island TS allows one train of Auxiliary Building Special Ventilation to be inoperable for 7 days. The requested change ensures that one train is operable.
 - The Shield Building High Range Ventilation Gas Radiation Monitors (R-50) powered by MCC 1T1 and 1T2 have no control function and not covered by the TS.
- Miscellaneous Receptacle and Distribution Panels:
 - MCC 1T1 powers Receptacle Panel 1RPA8 and Distribution Panel 119. MCC 1T2 powers Receptacle Panel 1RPB8 and Distribution Panel 219. Receptacle panels power some control room convenience receptacles and lighting and the R-50 radiation monitor sample pumps.
 - Panel 119 supplies the Train A Event Monitoring Room East Unit Cooler (there is another unit cooler supplied by Panel 135 in the Train A Event Monitoring Room). Panel 219 supplies the Train B Event Monitoring Room Unit Cooler (which is the only unit cooler in the Train B Event Monitoring Room). TS 3.15 would apply in the event room temperature exceeds the maximum to support equipment operability (89 °F). However, at most, the proposed change would affect one channel of event monitoring; the most limiting allowed out of service time for one channel of event monitoring equipment is 30 days.
 - Panels 119 and 219 each supply two unit coolers in the Relay Room. Should Relay Room temperature exceed the maximum limit for equipment operability (120 °F), instruments in TS Section 3.5 tables would be affected. However, current analysis indicates that the relay room would not heat up to the point of

affecting equipment operability (120 °F) unless all unit coolers are lost. The proposed change ensures at least two unit coolers will be operable in the Relay Room; therefore, elevated temperatures are not expected to affect equipment operability.

- Boric Acid Heat Trace Switchracks - MCC 1T1 powers switchracks I and III; MCC 1T2 powers switchracks II and IV. These components support boric acid heat trace. Per TS 3.2.C.4, one channel of heat tracing may be out of service for up to 72 hours. There are two channels of redundant heat tracing for each heat-traced pipe. The heat trace switchracks provide these redundant channels (with redundant channels for any given pipe segment being powered by redundant MCCs).
- Miscellaneous Cooling Water System Motor Valves:
 - MCC 1T1 powers MV-32322, "Loop A/B Cooling Water Return Header Crossover Motor Valve A," and MV-32144, "Loop A/B Cooling Water Header Crossover Motor Valve A," and MV-32038, "Cooling Water Dump Motor Valve." MCC 1T2 powers MV-32159, "Loop A/B Cooling Water Header Crossover Motor Valve B," and MV-32329, "Loop A/B Cooling Water Return Header Crossover Motor Valve B."

TS 3.3.D.2.b allows one cooling water header to be out of service for up to 72 hours, with certain provisions. MV-32144 and MV-32159 close to split the cooling water ring header into two trains upon receipt of a safety injection signal. MV-32144 and MV-32159 are redundant valves. Typically, if the power supply for either of these valves is to be taken out of service, the valve is closed (placed in its safeguards position) such that the cooling water header remains in service.

MV-32322, MV-32329, and MV-32038 are configured to provide an alternate return flow path should a normal return header become blocked. There is no separation requirement for the return header. These valves are currently maintained closed for operational convenience. Therefore, none of these valves is controlled by TS.

As described above, the staff has concluded that the proposed amendments will not lead to any of the equipment that is powered by either MCC 1T1 or MCC 1T2 being out of service for longer than currently allowed by TS. The amendments allow only one MCC (either MCC 1T1 or MCC 1T2) to be out of service at a time for no more than 72 hours for one time only. The opposite train MCC, its associated 480V buses, diesel generators, and safeguards equipment are verified to be operable thus ensuring that one train will be available during the transfer switch installation. Based on the above evaluation, the staff finds the proposed TS amendment acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (65 *FR* 43049). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Saba

Date: December 15, 2000