

July 13, 1987

Docket No. 50-286

Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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<u>Docket File</u>	E. Jordan
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C. Vogan	GPA/PA
M. Slosson	ARM/LFMB
OGC	J. Johnson
D. Hagan	

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. 78 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 6, 1987 (TAC 61096).

The amendment changes the Indian Point 3 Technical Specifications relative to the Reactor Vessel Level Indication System (RVLIS). The purpose of the proposed changes is to incorporate the appropriate limiting conditions for operation and the surveillance requirements for RVLIS. The installation of RVLIS is being implemented in accordance with the requirements of NUREG-0737, Item II.F.2 "Instrumentation for Detection of Inadequate Core Cooling." Editorial changes are also included.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 78 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

* SEE PREVIOUS CONCURRENCE

PDI-1
CVogan*
6/09/87

PDI-1
MSlosson*
6/10/87

OGC
MKarman*
6/11/87

rgc
PDI-1
RCapra
7/13/87

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PDR

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Senior Vice President - Nuclear Generation
Power Authority of the State of New York
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CVogan
6/9/87

PDI-1 *MS*
MSlosson
6/10/87

OGC-B
MYRON KAMON
MMS 6/11/87
PDI-1
RCapra
6/ /87

*OK per phone
conv. 6/16/87.
Concurrence left off
amend package. Concurrence
on NOTICE.*
*MMS
6/16/87*

Mr. John C. Brons
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of New York

Indian Point Nuclear Generating
Unit No. 3

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- 2 -

Indian Point 3

cc

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated March 6, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 13, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT NO. 78
FACILITY OPERATING LICENSE NO. DPR-64
DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

Table 3.5-5 (sheet 2 of 3)
Table 3.5-5 (sheet 3 of 3)
Table 4.1-1 (sheet 5 of 5)

Insert Pages

Table 3.5-5 (sheet 2 of 3)
Table 3.5-5 (sheet 3 of 3)
Table 4.1-1 (sheet 5 of 5)

TABLE 3.5-5 (Sheet 2 of 3)

PARAMETER	1 NO. OF CHANNELS AVAILABLE	2 MIN. NO. OF CHANNELS REQUIRED**	3 INDICATOR/ RECORDER**
15) Level Sensors in Lower Level of Turbine Building	2	1	ALARM
16) Reactor Coolant System Subcooling Margin Monitor	1	1	RECORDER
17) PORV Position Indicator (Acoustic Monitor)	1/Valve	1/Valve	INDICATOR
18) PORV Position Indicator (Limit Switch)	1/Valve	1/Valve****	INDICATOR AND ALARM
19) PORV Block Valve Position Indicator (Limit Switch)	1/Valve***	1/Valve	INDICATOR
20) Safety Valve Position Indicator (Acoustic Monitor)	1/Valve	1/Valve	INDICATOR
21) Auxiliary Feedwater Flow Rate	1/Pump	1/Pump	INDICATOR
22) Containment Water Level (Wide Range)	2	1	INDICATOR/ RECORDER
23) Containment Hydrogen Monitor	2	1	INDICATOR/ RECORDER
24) High-Range Containment***** Radiation Monitors (R25 R26)	2	1	ALARM
25) Core Exit Thermocouples	4/quadrant	2/quadrant	INDICATOR
26) Reactor Vessel Level Indication System (RVLIS)	2	1	INDICATOR

* One level channel per steam generator (either wide range or narrow range) with at least two wide range channels.

** Columns 2 and 3 may be modified to allow the instrument channel(s) to be inoperable for up to 7 days and/or the recorder to be inoperable for up to 14 days.

*** Except at times when valve operator control circuit is de-energized.

**** Except when the respective block valve is closed.

TABLE 3.5-5 (Sheet 3 of 3)

- ***** If the high-range containment radiation monitor is determined to be inoperable when the reactor is above the cold shutdown condition, then restore the monitoring capability within 7 days, and
- a) Initiate an alternate monitoring capability as soon as practical, but no later than 72 hours after identification of the failure of the monitor. If the monitor is not restored to operable status within 7 days, then,
 - b) Submit a Special Report to the NRC pursuant to Technical Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system.
- + If both narrow range analog monitor channels are determined to be inoperable, at least one channel will be restored to operable status within 30 days or the Plant will be brought to hot shutdown within the next 12 hours.

With the exception of the High Range Containment Radiation Monitors, if the minimum number of channels required are not restored to meet the above requirements within the time periods specified, then:

1. If the reactor is critical, it shall be brought to the hot shutdown condition utilizing normal operating procedures. The shutdown shall start no later than at the end of the specified time period.
2. If the reactor is subcritical, the reactor coolant system temperature and pressure shall not be increased more than 25°F and 100 psi, respectively, over existing values.
3. In either case, if the requirements of Columns 2 and 3 are not satisfied within an additional 48 hours, the reactor shall be brought to the cold shutdown condition utilizing normal operating procedures. The shutdown shall start no later than the end of the 48 hours period.

Table 4.1-1 (Sheet 5 of 5)

	<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
39.	High Range Containment Radiation Monitoring (R25, R26)	D	R	Q	
40.	Core Exit Thermocouples	D	N.A.	N.A.	
41.	Overpressure Protection System (OPS)	D	R	R	
42.	Reactor Vessel Level Indication System (RVLIS)	D	R	N.A.	

* To be effective after completion of all required modifications.

S -Each Shift

P -Prior to each startup if not done previous week

Q -Quarterly

NA -Not applicable

D -Daily

W -Weekly

M -Monthly

R -Each Refueling Outage



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

INTRODUCTION

By letter dated March 6, 1987, the Power Authority of the State of New York submitted proposed Technical Specifications concerning limiting conditions for operation and surveillance requirements for the Reactor Vessel Level Indication System (RVLIS).

EVALUATION AND DISCUSSION

Generic Letter 83-37 dated November 1, 1983 provided guidelines concerning NUREG-0737 "Clarification of TMI Action Plan Requirements" related to Technical Specifications. The proposed Technical Specifications contained in the licensee's March 6, 1987 submittal relate to the RVLIS which is being implemented in accordance with the requirements of NUREG-0737, Item II.F.2, "Instrumentation for Detection of Inadequate Core Cooling." RVLIS outputs are displayed on the plant Qualified Safety Parameter Display System. The system performs an input autocalibration sequence by automatically injecting test signals directly into every input on a regular schedule while the system is on line.

Previous revisions to the licensee's Technical Specifications added requirements for the subcooling margin monitor and the core exit thermocouples. The proposed Technical Specifications revise Tables 3.5-5 and 4.1-1 to incorporate the RVLIS requirements. Editorial changes are also being made to the Tables.

The proposed changes have been reviewed and found to be in accordance with the guidance in Generic Letter 83-37. The proposed changes are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that

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may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 13, 1987

PRINCIPAL CONTRIBUTOR:

T. Huang