

June 19, 1997

Mr. Stephen E. Quinn  
Vice President, Nuclear Power  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, NY 10511

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING UNIT NO. 2  
(TAC NO. M98405)

Dear Mr. Quinn:

The Commission has issued the enclosed Amendment No. 193 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated March 31, 1997.

The amendment revises TSs to remove the reference of Valve 863 from TS Table 3.6-1. This revision would allow for the installation of a proposed modification for automatic closure of Valve 863 upon receipt of a Phase A containment Isolation signal.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Jefferey F. Harold, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No.193 to DPR-26  
2. Safety Evaluation

cc w/encls: See next page

DOCUMENT NAME: G:\IP2\IP298405.AMD

\*See Previous Concurrence

Handwritten initials: JFH

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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The amendment revises TSs to remove the reference of Valve 863 from TS Table 3.6-1. This revision would allow for the installation of a proposed modification for automatic closure of Valve 863 upon receipt of a Phase A containment Isolation signal.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Jefferey F. Harold".

Jefferey F. Harold, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-247

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2. Safety Evaluation

cc w/encls: See next page

Stephen E. Quinn  
Consolidated Edison Company  
of New York, Inc.

Indian Point Nuclear Generating  
Station Units 1/2

cc:

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DATED: June 19, 1997

AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. DPR-26-INDIAN POINT UNIT 2

Docket File  
PUBLIC  
PDI-1 Reading  
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J. Harold  
OGC  
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W. Beckner  
ACRS  
J. Rogge, Region I  
C. Berlinger

cc: Plant Service list



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 31, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Alexander W. Dromerick, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 19, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

Insert Page

Table 3.6-1

Table 3.6-1

Table 3.6-1

Non-Automatic Containment Isolation Valves Open Continuously  
Or Intermittently for Plant Operation

3418	851A	SWN-44-5-A or B <sup>(1)</sup>	1814B
3419	850A	SWN-51-5 <sup>(1)</sup>	1814C
		SWN-44-1-A or B <sup>(1)</sup>	
4136	851B	SWN-51-1 <sup>(1)</sup>	5018
			5019
744	850B	SWN-44-2-A or B <sup>(1)</sup>	5020
		SWN-51-2 <sup>(1)</sup>	
888A	859A	SWN-44-3-A or B <sup>(1)</sup>	5021
888B	859C	SWN-51-3 <sup>(1)</sup>	5022
958			5023
959	3416	SWN-44-4-A or B <sup>(1)</sup>	5024
990D	3417	SWN-51-4 <sup>(1)</sup>	5025
1870	5459	SWN-71-5-A or B <sup>(1)</sup>	E-2
743	753H	SWN-71-1-A or B <sup>(1)</sup>	E-1
732	753G	SWN-71-2-A or B <sup>(1)</sup>	E-3
885A	SWN-41-5-A or B <sup>(1)</sup>	SWN-71-3-A or B <sup>(1)</sup>	E-5
885B	SWN-42-5	SWN-71-4-A or B <sup>(1)</sup>	MW-17
			MW-17-1
205	SWN-43-5	SA-24	85C
226	SWN-41-1-A or B <sup>(1)</sup>	SA-24-1	85D
227	SWN-42-1	PCV-1111-1	95C
250A	SWN-43-1	PCV-1111-2	95D
4925	SWN-41-2-A or B <sup>(1)</sup>	580A	
250B	SWN-42-2	580B	
4926	SWN-43-2	UH-43	
250C	SWN-41-3-A or B <sup>(1)</sup>	UH-44	
4927	SWN-42-3	990A	
250D	SWN-43-3	990B	
4928	SWN-41-4-A or B <sup>(1)</sup>	1814A	
869A	SWN-42-4		
878A	SWN-43-4		
869B			

(1) Either A or B valve(s) may serve as the required containment isolation valve(s) for the SWN-41, SWN-44 and SWN-77 series. Designation of the B valves(s) in the SWN-44 series requires the codesignation of the SWN-51 valve(s) associated with the penetration(s) as an additional required containment isolation valves(s).



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. DPR-26  
CONSOLIDATED EDISON COMPANY OF NEW YORK  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2  
DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated March 31, 1997, Consolidated Edison Company of New York (the licensee) requested an amendment to change the Technical Specifications (TSs) for Indian Point Nuclear Generating Unit No. 2 (IP2). The licensee proposed the amendment to remove containment isolation valve 863 from Table 3.6-1, Non-automatic Containment Isolation Valves Open Continuously Or Intermittently for Plant Operation. The removal of the valve from the table would allow a proposed modification for automatically closing upon receipt of a Phase A containment isolation signal.

2.0 EVALUATION

Valve 863 is a containment isolation valve for the backup nitrogen supply line to containment. Currently section 5.2 of the Updated Final Safety Analysis Report (UFSAR) describes valve 863 as normally closed during plant operation. However, TS Section 3.6, specifically Table 3.6.1, lists valve 863 as a non-automatic containment isolation valve which may be open continuously or open intermittently for plant operation. Presently, and in the past, valve 863 has been manually opened from the control room as needed to maintain adequate nitrogen pressure for safety-related and alternate safe shutdown equipment in containment.

To remedy the inconsistencies, the licensee has proposed to remove valve 863 from TS table 3.6-1. Further, upon approval, the valve will continue to be a manually controlled valve but will have an automatic closure signal. The fail-safe position of valve 863 will remain closed and the added containment isolation signal will de-energize the existing solenoid SOV-863. Upon de-energization, SOV-863 will exhaust air from valve 863 which will cause it to close. The present design of SOV-863 and valve 863 will be maintained. Only the control circuit would be changed to provide for the automatic containment isolation signal and to allow for reset after actuation of containment isolation. This will permit intentional post-accident manual actuation of valve 863 by operators, as required.

### 3.0 CONCLUSION

Based on the NRC staff review of the information submitted, the staff concludes that the licensee proposed revisions to IP2 TSs regarding the removal of valve 863 from TS Table 3.6-1 is acceptable. The licensee may incorporate the proposed change into the TSs for IP2.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (62 FR 26823). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Harold

Date: June 19, 1997