

AUGUST 11 1978

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Docket No. 50-286

Power Authority of the State
of New York
ATTN: Mr. George T. Berry
General Manager and
Chief Engineer
10 Columbus Circle
New York, New York 10019

Gentlemen:

The Commission has issued the enclosed Amendment No. 16 to Facility
Operating License No. DPR-64 for the Indian Point Nuclear Generating
Unit No. 3. The amendment consists of changes to the Technical
Specifications in response to your application dated August 8, 1978.

The amendment allows the licensee to apply the inspection techniques
and acceptance criteria of the Summer 1975 addenda to the 1974
Edition of Section XI of the ASME Code to weld L-1 in the Reactor
Coolant System Pressurizer.

Copies of the Safety Evaluation and the Notice of Issuance are also
enclosed.

Sincerely,

AS

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 16 to DPR-64
- 2. Safety Evaluation
- 3. Notice

cc w/encl:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 11, 1978

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Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No.16 to DPR-64
2. Safety Evaluation
3. Notice

cc w/encl:
See next page

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Region II Office
ATTN: EIS COORDINATOR
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New York, New York 10007



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 16
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated August 8, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64, is hereby amended and paragraph 2.C.(4) is added as follows:

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"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 16, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

(4) Pressurizer Weld Inspection

The results of the UT inspection of pressurizer weld L-1, (ref. Appendix A Technical Specification 4.2.5.f) shall be reported to the NRC and approval of the results obtained prior to return to power operation following the second refueling shutdown."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 11, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 16

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove

4.2-2
Sheet 4 of Table 4.2-1

Insert

4.2-2
Sheet 4 of Table 4.2-1

Marginal lines indicate where changes have been made.

4.2.4

The inspection interval shall be ten years.

4.2.5

The following definitions shall apply to the inspection methods employed in Table 4.2-1. The paragraphs referenced are corresponding paragraphs of Section XI of the ASME Code for In-Service Inspection of Nuclear Reactor Coolant Systems dated January 1970, except as noted:

- a. UT - Ultrasonic examination per paragraph IS-213.2.*
- b. RT - Radiographic examination per paragraph IS-213.1.
- c. MT - Magnetic particle examination per paragraph IS-212.1.
- d. PT - Liquid penetrant examination per paragraph IS-212.2.
- e. V - Visual examination per paragraphs IS-211.1 or IS-211.2.

* All indications which produce a response greater than 100% of the reference level shall be investigated to the extent that the operator can determine the shape, identity, and location of all such reflectors and evaluate these indications per paragraph IS-311.

- f. Weld L-1 in the pressurizer shall utilize the inspection techniques and acceptance criteria of the Summer 1975 addenda to the 1974 Edition of Section XI of the ASME Code.

4.2.6

Examinations which reveal unacceptable structural defects in a category shall be extended to include an additional number (or areas) of system components or piping in the same category approximately equal to that initially examined. In the event further unacceptable structural defects are revealed, all remaining system components or piping in the category shall be examined to the extent specified in that examination category.

4.2.7

With the exception of those components or areas for which the examination may be deferred to the end of the inspection interval, at least 25 percent of the required examinations shall have been completed by

TABLE 4.2-1 (Sheet 4 of 12)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
		PRESSURIZER			outages shall be performed during each refueling period. Where access to the space below the reactor core during normal refueling outages precludes inspection of this space, at least one examination, at or near the end of each inspection interval, shall be conducted under conditions which enable inspection.
2.1	B	Longitudinal and circumferential welds.	V & UT	Longitudinal - 10% Circumferential - 5%	Accessibility of welds is limited by biological and missile shield. Longitudinal weld L1 shall be inspected during the second refueling outage using straight beam, 45° and 60° half-angle beam UT methods for its accessible length.
2.2	D	Nozzle-to-vessel welds	V & UT	See remarks	Instrument and sample nozzles are included in Item 2.4.
2.3	E-1	Heater connections		See remarks	These connections are considered in Item 2.4.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 16 TO FACILITY LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT UNIT NO. 3

DOCKET NO. 50-286

Introduction

By letter dated August 8, 1978, the Power Authority of the State of New York (the licensee) requested amendment of Technical Specifications appended to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment would augment the inspection requirements of the reactor coolant system pressurizer. We have made certain changes to the proposed amendment. These have been discussed with and accepted by the licensee.

Background

During the current 1978 refueling outage, ultrasonic examinations by 40° angle beam was performed in accordance with the 1970 Edition of ASME Section XI Code on longitudinal weld L-1 in the reactor coolant system pressurizer. No recordable indications were noted. This weld was also ultrasonically examined utilizing a straight beam. An indication approximately 4.1 inches long by .5 inches wide was detected. Further tests indicated that it was laminar in nature and approximately 2 inches below the outside surface of the weld. The wall thickness at the location of the indication was reported by the licensee to be approximately 4 inches. Radiographs of the as-fabricated weld and baseline ultrasonic testing data (angle beam in accordance with the 1970 Section XI Code) for this weld contained no reportable indications.

Licensee's Proposed Changes

Weld L-1 in the pressurizer will be inspected during the second refueling outage (1979-80) using straight beam, 45°, and 60° half-node angle UT methods and acceptance criteria of the 1974 Edition of Section XI Code up to Summer 1975 Addenda.

Licensee's Basis for the Proposed Changes

The licensee states that the 1970 Edition of Section XI Code has no acceptance parameters for such an indication whereas the Summer 1975 Addenda of Section XI provides acceptance parameters for both planar and laminar indications. Based on the acceptance standards of Tables IWB 3511.1 and IWB 3511.3 of the 1974 Edition of Section XI Code up to Summer '75 Addenda, licensee concludes that the indication found in L-1 weld by straight beam examination would be acceptable to either of these tables regardless of whether the indication is characterized as planar or laminar. The licensee also concludes that the indication was present as-fabricated, however, because of its orientation and almost insignificant thickness it would not be expected to appear on the radiographs.

Evaluation

We agree with the licensee's conclusions that the indication was present as-fabricated, not service-induced. This component is very unlikely to experience service conditions which would cause cracking. Furthermore, because of its orientation and no measurable through wall dimension it would not be expected to appear on the radiographs of the as-fabricated weld or to be revealed in the baseline angle beam UT results. We also agree that this indication could be characterized and evaluated in accordance with the NRC endorsed 1974 Edition of Section XI plus Addenda up to Summer '75 Addenda and is acceptable.

In summary, we conclude that the indication detected in L-1 weld is acceptable and the structural integrity of the affected pressurizer will not be compromised by further operation. We further conclude that the licensee's proposed changes to Technical Specification §4.2.5 are also acceptable. The UT results of the reinspection of the L-1 weld during the next refueling outage will be reported to and approved by the Commission before IP-3 is returned to power operation. This requirement, although not proposed in the application, was discussed with and agreed to by the licensee.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 11, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-286POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 16 to Facility Operating License No. DPR-64, issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3, located in Buchanan, Westchester County, New York. The amendment is effective as of its date of issuance.

The amendment allows the licensee to apply the inspection techniques and acceptance criteria of the Summer 1975 Addenda to the 1974 Edition of Section XI of the ASME Code to weld L-1 in the Primary Coolant System Pressurizer and finds the indication in that weld acceptable in accordance with those criteria.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazard's consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 8, 1978, (2) Amendment No. 16 to License No. DPR-64, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 11th day of August 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors