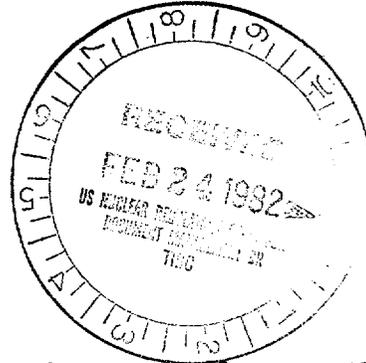


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Docket No. 50-286

Mr. George T. Berry, President  
and Chief Operating Officer  
Power Authority of the State of New York  
10 Columbus Circle  
New York, New York 10019



Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 42 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consist of changes to the Technical Specifications in response to your application transmitted by letter dated January 6, 1982.

The amendment revises the provisions in the Technical Specifications to allow the reorganization of the Indian Point Unit No. 3 Safety Review Committee. In essence, personnel titles and reporting requirements have been changed in order to clarify and unify responsibilities. Using various tools, such as the Task Action Plan, Regulatory Guide 1.33 and ANSI 18.7/ANS 3.2 guidelines we reviewed the Indian Point Unit No. 3 management organization to assure that the proposed change does not diminish the level of safety with respect to organization and technical resources provided by the existing Technical Specifications.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

The amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and, therefore, does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

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Mr. George T. Berry

- 2 -

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original signed by:  
S. A. Varga  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

- 1. Amendment No. 42 to DPR-26
- 2. Notice of Issuance

cc w/enclosures:  
See next page

FR NOTICE  
&  
AMENDMENT

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SURNAME	CParrish	JThomas	SVarga	TNowak	KARMAN		
DATE	02/1/82	02/4/82	02/1/82	02/5/82	02/10/82		

Mr. George T. Berry  
Power Authority of the State of New York

cc: White Plains Public Library  
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Indian Point 3 Nuclear Power Plant  
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Buchanan, New York 10511

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Mayor, Village of Buchanan  
188 Westchester Avenue  
Buchanan, New York 10511

Mr. J. P. Bayne, Senior Vice Pres.  
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Resident Inspector  
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U. S. Nuclear Regulatory Commission  
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Ezra I. Bialik  
Assistant Attorney General  
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New York State Department of Law  
2 World Trade Center  
New York, New York 10047



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated January 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

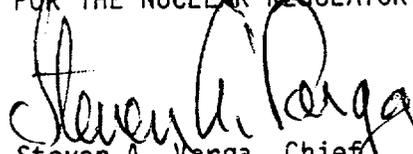
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 17, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

5-8

6-2

6-9

6-11

6-12

Insert Pages

5-8

6-2

6-9

6-11

6-12

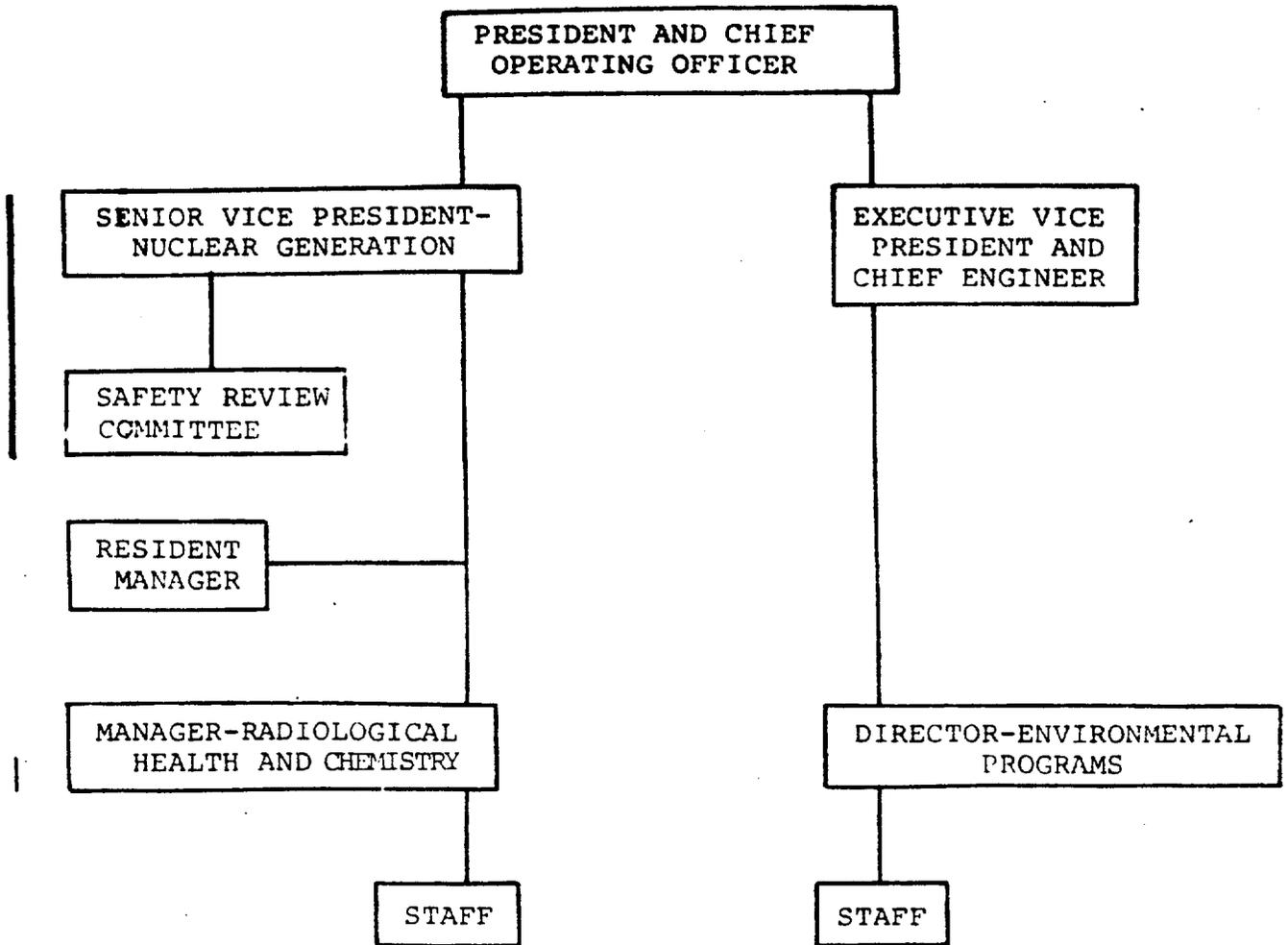
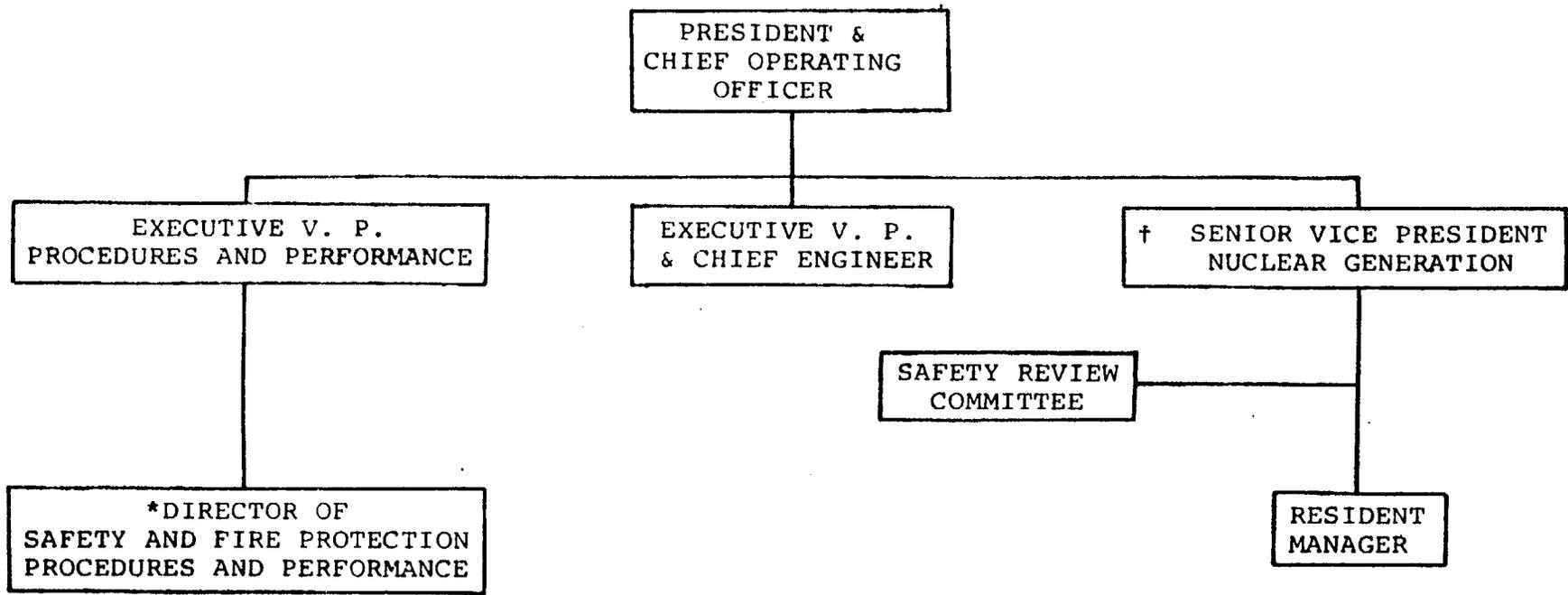


FIGURE 5.2-2  
 MANAGEMENT ORGANIZATION-OFFSITE ENVIRONMENT  
 INDIAN POINT 3 NUCLEAR POWER PLANT

Amendment No. 39 42



†The Senior Vice President Nuclear Generation is responsible for administering the fire protection program.

\*The Director of Safety and Fire Protection is the off-site management position responsible for assessing the effectiveness of the fire protection programs by conducting reviews and audits.

FIGURE 6.2-1

MANAGEMENT ORGANIZATION CHART  
INDIAN POINT 3 NUCLEAR POWER PLANT

MEMBERSHIP

6.5.2.2. The SRC shall be composed of the following voting members:

Chairman:	Manager-Nuclear Safety Evaluation
Vice-Chairman:	Vice President-Quality Assurance
Member:	Vice President Nuclear Support-EMR
Member:	Vice President Nuclear Support-PCR
Member:	Manager-Radiological Health and Chemistry
Member:	Director-Nuclear Design and Analysis
Member:	Director-Electrical Design and Analysis
Member:	Director of Environmental Programs
Member:	Director-Civil/Structural Design and Analysis
Member:	Director-Mechanical Design and Analysis
Member:	Director-Piping and Process-Design and Analysis
Member:	Manager Operational Analysis and Training

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and four members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.

## AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the Senior Vice President-Nuclear Generation.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. A fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

## AUTHORITY

6.5.2.9 The SRC shall report to and advise the Senior Vice President-Nuclear Generation on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

## RECORDS

- 6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:
- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Generation within 14 days after the date of the meeting.
  - b. Reports of reviews encompassed by Section 6.5.2.7 above shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Generation within 14 days following completion of the review.
  - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President-Nuclear Generation and to the management positions responsible for the areas audited within 30 days after the completion of the audit.

## CHARTER

- 6.5.2.11 Conduct of the committee will be in accordance with a charter, approved by the Senior Vice President-Nuclear Generation, setting forth the mechanism for implementation of the committee's responsibilities and authority.

## 6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:
- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
  - b. Each REPORTABLE OCCURRENCE requiring 24 hours notification to the Commission shall be reviewed by the PORC and a report submitted by the Resident Manager to the Chairman of the SRC and Senior Vice President-Nuclear Generation.

## 6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36(c)(1)(i).
  - b. The Safety Limit violation shall be reported immediately to the Commission. The Chairman of the SRC and Senior Vice President-Nuclear Generation will be notified within 24 hours.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-286POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 42 to Facility Operating License No. DPR-64, issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective as of the date of issuance.

The amendment revises the provisions in the Technical Specifications to allow the reorganization of the Indian Point Unit No. 3 Safety Review Committee.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

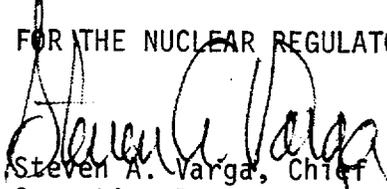
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated January 6, 1982, (2) Amendment No. 42 to License No. DPR-64, and (3) the Commission's letter to the licensee dated February 17, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of February, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing