

September 8, 1989

Docket No. 50-286

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Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 71416)

The Commission has issued the enclosed Amendment No. 89 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated November 10, 1988, as supplemented July 24, 1989.

The amendment revises the Technical Specifications to eliminate organization charts, Figures 6.2-1 and 6.2-2.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Joseph D. Neighbors, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 89 to DPR-64
- 2. Safety Evaluation

cc: w/enclosures
See next page

[AMEND 71416]

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Indian Point 3 Nuclear Power Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 89
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated November 10, 1988, as supplemented July 24, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 89, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 8, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 89

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

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Amendment No. 14, §7

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Resident Manager shall be responsible for overall facility operation. During periods when the Resident Manager is unavailable, the Superintendent of Power will assume his responsibilities. In the event both are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel. The Resident Manager reports directly to the Executive Vice President-Nuclear Generation.

6.2 ORGANIZATION

6.2.1 Facility Management and Technical Support

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a) Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR.
- b) The Resident Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c) The Executive Vice President - Nuclear Generation shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

- d) The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 PLANT STAFF

- a) Each duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b) At least one Licensed Operator shall be in the control room when fuel is in the reactor.
- c) At least two Licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d) An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e) ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f) A Fire Brigade of at least five members shall be maintained on site at all times. This excludes four members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency. During periods of cold shutdown the Fire Brigade will exclude two members of the minimum shift crew.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the unit is operating. (Operating personnel are defined as on shift senior reactor operators, reactor operators, nuclear plant operators, shift technical advisors and shift contingency health physicists, I&C and maintenance personnel.) However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification on a temporary basis the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Resident Manager or his deputy, or higher levels of management, in accordance with established procedures.

- h) At least one individual holding a Senior Reactor Operator (SRO) license shall be on duty in the Control Room at all times.
- i) The Operations Superintendent, Shift Supervisor and Senior Reactor Operator shall hold a Senior Reactor Operator (SRO) license. The Reactor Operator shall hold a reactor operator (RO) license.

6.3 PLANT STAFF QUALIFICATIONS

6.3.1 Each member of the plant staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Radiation and Environmental Services Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the training coordinator and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Training Coordinator and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976 with the exception of the training program schedule.

6.4.3 A training program for use of the post-accident sampling system shall be maintained to ensure that the plant has the capability to obtain and analyze reactor coolant and containment atmosphere samples under post-accident conditions.

6.4.4 A training program shall be maintained to ensure that the plant has the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluent during and following an accident.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATING REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The Plant Operating Review Committee shall function to advise the Resident Manager on all matters related to nuclear safety and all matters which could adversely change the plants environmental impact.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 89 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

INTRODUCTION

By letter dated November 10, 1988, as supplemented July 24, 1989, the Power Authority of the State of New York (the licensee) proposed changes to Technical Specifications (TS) 6.2.1, Offsite Organization, and 6.2.2, Plant Staff. The proposed changes would remove Figure 6.2-1, Management Organization Chart, and Figure 6.2-2, Plant Staff Organization, and replace them with a narrative description of the offsite and onsite organizations functional requirements in TS 6.2.1 and plant staff qualifications in 6.2.2. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter 88-06, dated March 22, 1988.

The licensee's letter dated July 24, 1989 provided a revised Technical Specification page 6-3. The page relocated the requirement for the Operations Superintendent to have an SRO license. This requirement was previously on Figure 6.2-2. This page did not add any new requirements. Therefore this page was not noticed as it did not substantively alter the request or affect the initial determination published in the Federal Register on January 11, 1989.

BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Specifications 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Because of these limitations on organizational structure, the nuclear industry has highlighted this as an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept that included the endorsement of the proposed changes by the Westinghouse Owners Group. In its review of the Shearon Harris proposal, the

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staff concluded that most of the essential elements of offsite and onsite organization charts are captured by other regulatory requirements, notably, Appendix B to 10 CFR 50. However, there were aspects of the organizational structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organization charts. The applicable regulatory requirements are those administrative controls that are necessary to ensure safe operation of the facility. Therefore, those aspects of organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to Specification 6.2.1 to define functional requirements for the offsite and onsite organizations and (2) to Specification 6.2.2 to define qualification requirements of the plant staff.

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NFP-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from TS. Generic Letter 88-06 provided this guidance to all power reactors.

EVALUATION

The licensee's proposed changes to its TS are in accordance with the guidance provided by Generic Letter 88-06 and addressed the items listed below:

- (1) Specifications 6.2.1 and 6.2.2 were revised to delete the references to Figures 6.2-1 and 6.2-2 that were removed from the TS.
- (2) Functional requirements of the offsite and onsite organizations were defined and added to Specification 6.2.1, and they are consistent with the guidance provided in Generic Letter 88-06. The specification notes that implementation of these requirements is documented in the updated FSAR.
- (3) The senior reactor operator and reactor operator license qualified positions of the plant staff were added to Specification 6.2.2. Therefore, the requirement that was identified on the organization chart for the plant staff will be retained.
- (4) Consistent with requirements to document the offsite and onsite organization relationships in the form of organization charts, the licensee has confirmed that this documentation has been designated for inclusion to the annual update of the FSAR.

On the basis of its review of the above items, the staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in Generic Letter 88-06. Accordingly, the staff finds the proposed changes to be acceptable.

In addition to the organization changes, several editorial changes are made to the List of Figures. These changes were previously inadvertently overlooked. Figures 2.1-2 and 3.10-5 were deleted by Amendment No. 48, Figure 4.2-2 was deleted by Amendment No. 57 and Figure 3.8.1 was added by Amendment No. 70.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, or administrative procedures or requirements. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 8, 1989

PRINCIPAL CONTRIBUTORS:

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