REGULATORY DOCKET FILE COPY

January 19, 1980

Docket No. 50-286

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Mr. George T. Berry, Executive Director
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 28 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated August 24, 1979.

The amendment revises the reactor coolant system pressure and temperature heatup and cooldown curves and adds a requirement to conduct surveillance for core reactivity anomalies.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 28 to DPR-64
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

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OFFICE	DOR:ORB1	DOR:ORB1		DOR:ADOR	OELD	DOR:ORB1
SURNAME	LNOlshan:jb	CSParrish		WPGammill	S H Lewis	ASchwencer
DATE	12/10/79	12/19/79		1/4/80	1/9/80	1/2/79

#1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 19, 1980

Docket No. 50-286

Mr. George T. Berry, Executive Director
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 28 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated August 24, 1979.

The amendment revises the reactor coolant system pressure and temperature heatup and cooldown curves and adds a requirement to conduct surveillance for core reactivity anomalies.

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Sincerely,

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

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See next page

cc: White Plains Public Library
100 Martine Avenue
White Plains, New York 10601

Mr. Vito J. Cassan
Assistant General Counsel
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Anthony Z. Roisman
Natural Resources Defense Council
917 - 15th Street, N.W.
Washington, D. C. 20005

Dr. Lawrence R. Quarles
Apartment 51
Kendal at Longwood
Kennett Square, Pennsylvania 19348

Mr. George M. Wilverding
Licensing Supervisor
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Mr. P. W. Lyon
Manager - Nuclear Operations
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Director, Technical Development
Programs
State Of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. J. P. Bayne, Resident Manager
Indian Point 3 Nuclear Power Plant
P. O. Box 215
Buchanan, New York 10511

Mr. J. W. Blake, Ph.D., Director
Environmental Programs
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Theodore A. Rebelowski
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

Joyce P. Davis, Esquire
Law Department
Consolidated Edison Company of
New York Inc.
4 Irving Place
New York, New York 10003

Jeffrey C. Cohen, Esquire
New York State Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223

Honorable George Begany
Mayor, Village of Buchanan
188 Westchester Avenue
Buchanan, New York 10511

Ms. Ellyn Weiss
Sheldon, Harmon and Weiss
1725 I Street, N.W.
Suite 506
Washington, D. C. 20006

cc: Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Power Authority of the State of New York (the licensee) dated August 24, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-64 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 28, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 28

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages	Insert Pages
3.1-4	3.1-4
3.1-5	3.1-5
3.1-9	3.1-9
3.1-10	3.1-10
3.1-11	3.1-11
3.10-8	3.10-8
3.10-8a	3.10-8a
4.3-1	4.3-1
4.3-2	4.3-2
4.3-3	4.3-3

B. HEATUP AND COOLDOWN

Specifications

1. The reactor coolant temperature and pressure and system heatup and cooldown rates averaged over one hour (with the exception of the pressurizer) shall be limited in accordance with Figure 3.1-1 and Figure 3.1-2 for the service period up to 9.26 effective full-power years (EFPYs). The heatup or cooldown rate shall not exceed 100°F/hr.

   a. Allowable combinations of pressure and temperature for specific temperature change rates are below and to the right of the limit lines shown. Limit lines for cooldown rates between those presented may be obtained by interpolation.

   b. Figure 3.1-1 and Figure 3.1-2 define limits to assure prevention of non-ductile failure only. For normal operation other inherent plant characteristics, e.g., pump heat addition and pressurizer heater capacity may limit the heatup and cooldown rates that can be achieved over certain pressure-temperature ranges.

2. The limit lines shown in Figure 3.1-1 and Figure 3.1-2 shall be recalculated periodically using methods discussed in the Basis and results of surveillance specimens as covered in Specification 4.2. The order of specimen removal may be modified based on the results of testing of previously removed specimens.

3. The secondary side of the steam generator shall not be pressurized above 200 psig if the temperature of the steam generator is below 70°F.

4. The pressurizer heatup and cooldown rates averaged over one hour shall not exceed 100°F/hr and 200°F/hr, respectively. The spray shall not be used if the temperature difference between the pressurizer and the spray fluid is greater than 320°F.

5. Reactor Coolant System integrity tests shall be performed in accordance with Section 4.3.

Basis

Fracture Toughness Properties

The fracture toughness properties of the ferritic materials in the

reactor vessel are determined in accordance with the Summer 1965 Section III of the ASME Boiler and Pressure Vessel Code [6] and ASTM E185[5] and in accordance with additional reactor vessel requirements. These properties are then evaluated in accordance with Appendix G of the 1972 Summer Addenda to Section III of the ASME Boiler and Pressure Vessel Code [1], and the calculation methods described in WCAP-7924 [2].

The first reactor vessel material surveillance capsule was removed during the 1978 refueling outage. This capsule has been tested by Westinghouse Corporation and the results have been evaluated and reported [7]. Based on the Westinghouse evaluation, heatup and cooldown curves (Figures 3.1-1 and 3.1-2) were developed for up to 9.26 EFPYs of reactor operation.

The maximum shift in RT_{NDT} after 9.26 EFPYs of operation is projected to be 127°F at the 1/4 T and 85°F at the 3/4 T vessel wall locations, for Plate B2803-3 the controlling plate. Plate B2803-3 was also the controlling plate for the first operating period of 2 EFPYs.

Heatup and cooldown limit curves are calculated using the most limiting value of RT_{NDT} at the end of 9.26 years of service life. The 9.26 year service life period is chosen such at the limiting RT_{NDT} at the 1/4 T location in the core region is higher than the RT_{NDT} of the limiting unirradiated material. This service period assures that all components in the Reactor Coolant System will be operated conservatively in accordance with Code recommendations.

The highest RT_{NDT} of the core region material is determined by adding the radiation induced ΔRT_{NDT} for the applicable time period to the original RT_{NDT} shown in Table Q4.2-1 [3]. The fast neutron (E > 1 Mev) fluence at 1/4 thickness and 3/4 thickness vessel locations is given as a function of full power service life in Figure A-2 [7]. Using the applicable fluence at the end of the 9.26 year period and the copper content of the material in question, the ΔRT_{NDT} is obtained from Figure 4.4-3 [4].

REFERENCES:

1. ASME Boiler and Pressure Vessel Code, Section III, 1972 Summer Addenda.

2. WCAP-7924, "Basis for Heatup and Cooldown Limit Curves", W. S. Hazelton, S. L. Anderson, S. E. Yanichko, July 1972.

3. FSAR Volume 5, Response to Question Q4.2.

4. FSAR Section 4.

5. ASTM E185-70, Surveillance Tests on Structural Materials in Nuclear Reactors.

6. ASME Boiler and Pressure Vessel Code, Section III, Summer 1965.

7. WCAP-9491, "Analysis of Capsule T from the Indian Point Unit No. 3 Reactor Vessel Radiation Surveillance Program", J.A. Davidson, S.L. Anderson, W.T. Kaiser, April 1979.

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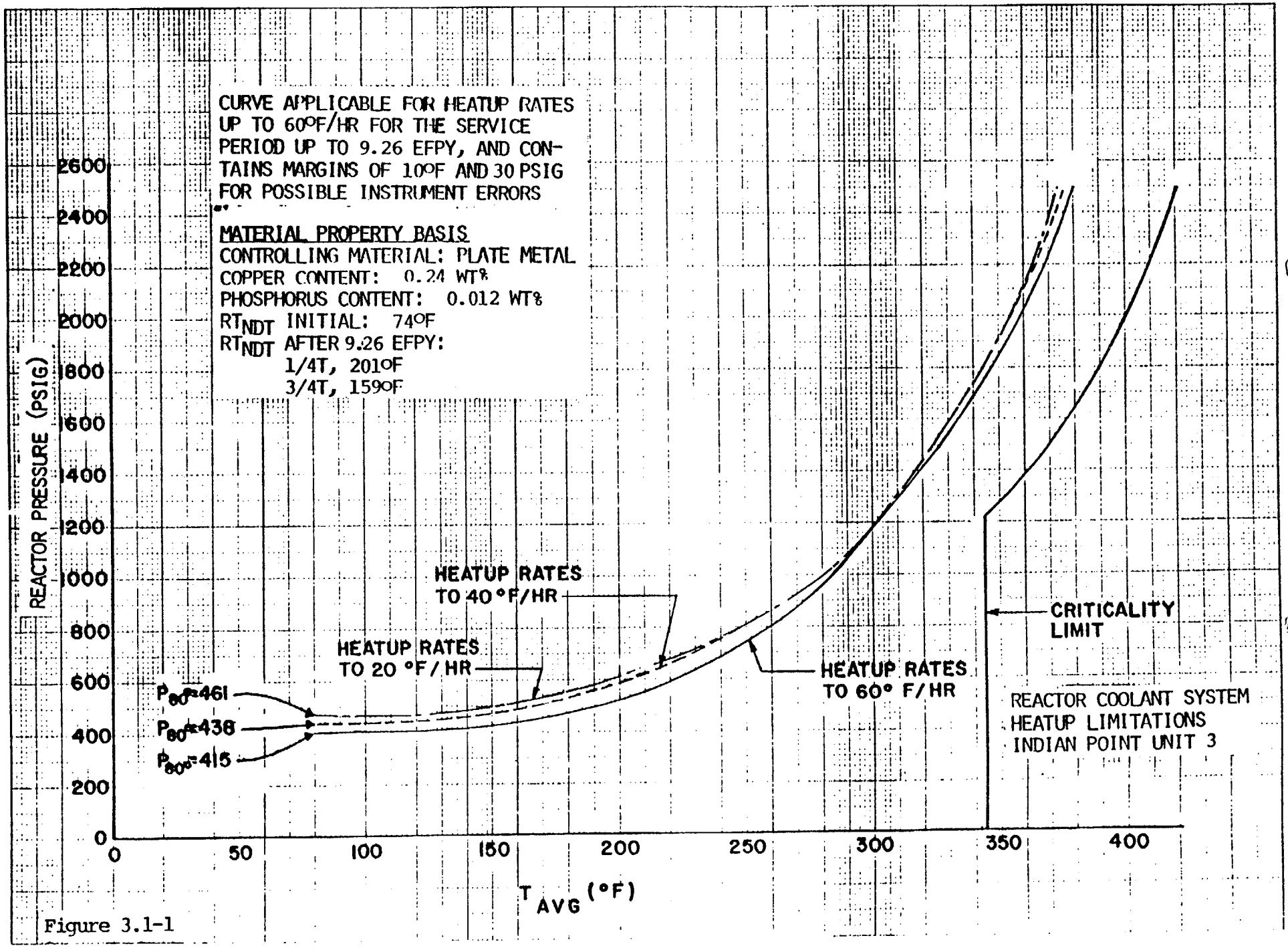

Figure 3.1-1

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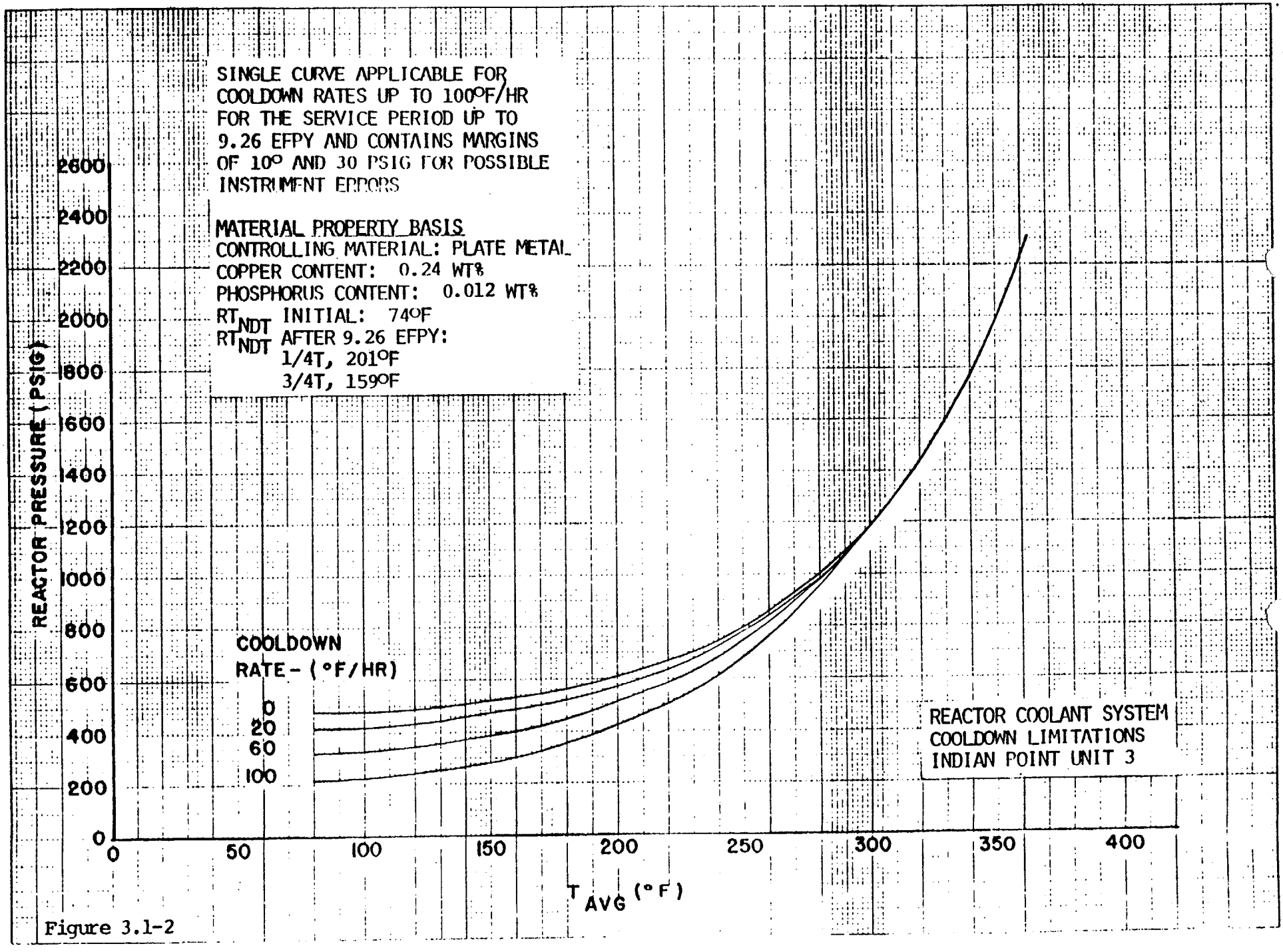

Figure 3.1-2

3.10.9 Rod Position Monitor

If the rod position deviation monitor is inoperable, individual rod positions shall be logged once per shift and after a load change greater than 10 percent of rated power.

3.10.10 Reactivity Balance

The overall core reactivity balance shall be compared to predicted values to demonstrate agreement within $\pm$ 1% $\Delta k/k$ at least once per 31 Effective Full Power Days (EFPD). This comparison shall, at least, consider reactor coolant system boron concentration, control rod position, reactor coolant system average temperature, fuel burnup based on gross thermal energy generation, xenon concentration, and samarium concentration. The predicted reactivity values shall be adjusted (normalized) to correspond to the actual core condition prior to exceeding a fuel burnup of 60 EFPD after each fuel loading.

3.10.11 Notification

Any event requiring plant shutdown or trip setpoint reduction because of Specification 3.10 shall be reported to the Nuclear Regulatory Commission within 30 days.

BASIS

Design criteria have been chosen for normal operations, operational transients and those events analyzed in FSAR Section 14.1 which are consistent with the fuel integrity analyses. These relate to fission gas release, pellet temperature and cladding mechanical properties. Also, the minimum DNBR in the core must not be less than 1.30 in normal operation or in short term transients.

In addition to the above conditions, the peak linear power density must not exceed the limiting Kw/ft values which result from the large break loss of coolant

accident analysis based on the ECCS acceptance criteria limit of 2200°F. This is required to meet the initial conditions assumed for loss of coolant accident analyses. To aid in specifying the limits on power distribution, the following hot channel factors are defined.

$F_Q(Z)$, Height Dependent Heat Flux Hot Channel Factor, is defined as the maximum local heat flux on the surface of a fuel rod at core elevation Z divided by the average fuel rod heat flux, allowing for manufacturing tolerances on fuel pellets and rods.

4.3 REACTOR COOLANT SYSTEM INTEGRITY TESTING

Applicability

Applies to test requirements for Reactor Coolant System integrity.

Objective

To specify tests for Reactor Coolant System integrity after the system is closed following normal opening, modification or repair.

Specification

a) When the Reactor Coolant System is closed after it has been opened, the system will be leak tested at not less than 2335 psig and in accordance with NDT requirements for temperature.

b) When Reactor Coolant System modifications or repairs have been made which involve new strength welds on components, the new welds will meet the requirements of ASME Section XI, IS400 and IS500.

c) The reactor coolant system leak test temperature-pressure relationship shall be in accordance with the limits of Figure 4.3-1 for heatup for the first 9.26 EFPY's of operation. Figure 4.3-1 will be recalculated periodically. Allowable pressures during cooldown from the leak test temperature shall be in accordance with Figure 3.1-2.

Basis

For normal opening, the integrity of the system, in terms of strength, is unchanged. If the system does not leak at 2335 psig (Operating pressure + 100 psi: ± 100 psi is normal system pressure fluctuation), it will be leak tight during normal operation.

For repairs on components, the thorough non-destructive testing gives a very high degree of confidence in the integrity of the system, and will detect any significant defects in and near the new welds. In all cases, the leak test will assure leak tightness during normal operation.

The inservice leak test temperatures are shown on Figure 4.3-1. The temperatures are calculated in accordance with ASME Code Section III, Appendix G. This Code requires that a safety factor of 1.5 times the stress intensity factor caused by pressure be applied to the calculation.

For the first 9.26 effective full power years, it is predicted that the highest RT_{NDT} in the core region taken at the 1/4 thickness will be 201°F. The temperature determined by methods of ASME Code Section III for 2335 psig is 135°F above this RT_{NDT} and for 2[illegible] psig (maximum) is 145°F above this RT_{NDT}. The minimum inservice leak test temperature requirements for periods up to ten effective full power years are shown on Figure 4.3-1.

The heatup limits specified on the heatup curve, Figure 4.3-1, must not be exceeded while the reactor coolant system is being heated to the inservice leak test temperature. For cooldown from the leak test temperature, the limitations of Figure 3.1-1 must not be exceeded. Figures 4.3-1 and 3.1-2 are recalculated periodically, using methods discussed in the Basis for Specification 3.1.B and results of surveillance specimens, as covered in Specification 4.2.

<u>Reference</u>

1. FSAR, Section 4.

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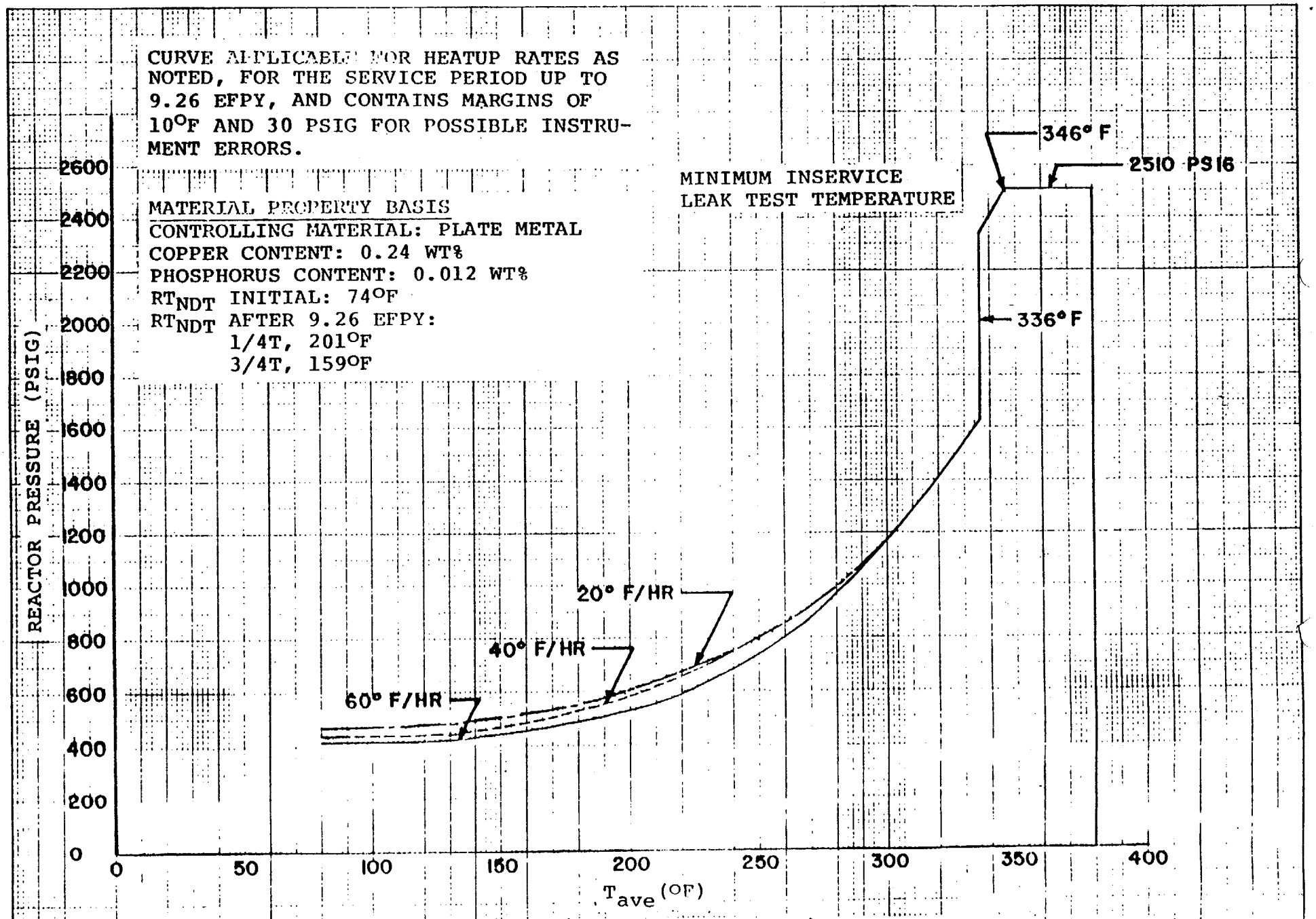

Figure 4.3-1 PRESSURE/TEMPERATURE LIMITATIONS FOR HYDROSTRATIC LEAK TEST

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

Introduction

By letter dated August 24, 1979, Power Authority of the State of New York (the licensee), submitted an application to amend the Technical Specifications appended to Facility Operating License DPR-64 for Indian Point Unit 3. One of the requested changes would modify the reactor vessel metal reference nil-ductility temperature (RT_{NDT}) due to irradiation. Another requested change would add a requirement to conduct surveillance for reactivity core anomalies.

Discussion

10 CFR Part 50, Appendix G "Fracture Toughness Requirements," requires that pressure-temperature limits be established for reactor coolant system heatup and cooldown operations, inservice leak and hydrostatic tests, and reactor core operation. These limits are required to ensure that the stresses in the reactor vessel remain within acceptable limits. They are intended to provide adequate margins of safety during any condition of normal operation, including anticipated operational occurrences.

The pressure-temperature limits depend upon the metallurgical properties of the reactor vessel materials. The properties of materials in the vessel beltline region vary over the lifetime of the vessel because of the effects of neutron irradiation. One principle effect of the neutron irradiation is that it causes the vessel material nil-ductility temperature (RT_{NDT}) to increase with time. The pressure-temperature operating limits must be modified periodically to account for this radiation induced increase in RT_{NDT} by increasing the temperature required for a given pressure. The operating limits for a particular operating period are based on the material properties at the end of the operating period. By periodically revising the pressure-temperature limits to account for radiation damage, the stresses and stress intensities in the reactor vessel are maintained within acceptable limits.

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The magnitude of the shift in RT_{NDT} is proportional to the neutron fluence that the materials are subjected to. The shift in RT_{NDT} can be predicted from Regulatory Guide 1.99. To check the validity of the predicted shift in RT_{NDT}, a reactor vessel material surveillance program is required. Surveillance specimens are periodically removed from the vessel and tested. The results of these tests are compared to the predicted shifts in RT_{NDT}, and the pressure-temperature operating limits are revised accordingly.

Although the licensee has been conducting surveillance for core reactivity anomalies, no requirement to do so existed in the Technical Specification. The omission is corrected by the proposed amendment.

Evaluation

The present Technical Specifications contain pressure-temperature operating limits that are effective for two EFPY. Since Indian Point 3 has operated for about two EFPY, Power Authority of the State of New York has submitted revised operating limits. These limits are proposed for operation through 9.26 EFPY. Radiation damage on vessel materials is predicted from the results of tests on material surveillance capsule T which was removed from the reactor vessel at 1.34 EFPY. The limiting material is lower shell plate metal having an initial RT_{NDT} of 74°F. At 9.26 EFPY the RT_{NDT} of the limiting material at the 1/4 T* location is calculated to increase to 201°F.

Power Authority of the State of New York transmitted copies of the surveillance report on Capsule T, WCAP-9491, with their application for the Technical Specification change. Specimens received an average fluence of 2.92×10^{18} n/m^2. This fluence resulted in an increase in RT_{NDT} for weld metal of 175°F and for plate material of 125°F. We have reviewed the test results on Capsule T and consider them to be valid.

We have reviewed the proposed pressure-temperature operating limits and have performed independent calculations to verify their compliance with Appendix G, 10 CFR Part 50. We agree that the limiting material is plate material having an RT_{NDT} of 201°F at 9.26 EFPY. We conclude that the proposed operating limits are acceptable and are in conformance with Appendix G for operation through 9.26 EFPY.

*1/4 T is one-fourth the thickness of the vessel wall, measured from the inside.

Conformance with Appendix G to 10 CFR Part 50 in establishing safe operating limitations will ensure adequate safety margins during operation, testing, maintenance, and postulated accident conditions and constitutes an acceptable basis for satisfying the requirements to NRC General Design Criterion 31, Appendix A, 10 CFR Part 50.

We find the proposed Technical Specification for conducting surveillance for core activity anomalies to be acceptable. The added paragraph 3.10.10 is identical to that in the Westinghouse Standard Technical Specifications.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 19, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-286

POWER AUTHORITY OF THE STATE OF NEW YORK

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 28 to Facility Operating License No. DPR-64, issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective as of the date of issuance.

The amendment revises the reactor coolant system pressure and temperature heatup and cooldown curves and adds a requirement to conduct surveillance for core reactivity anomalies.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 24, 1979, (2) Amendment No. 28 to License No. DPR-64, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 19th day of January, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Docket File

JULY 11 1979

Docket No. 50-286

REGULATORY DOCKET FILE COPY

Mr. George T. Berry, Executive Director
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 12, 1979.

The amendment revises the Environmental Technical Specifications (ETS) to consolidate all references to thermal plume mapping into Section 4.1.1.a, which, in turn, refers to the conditions of the New York State Certification issued by the Department of Environmental Conservation (DEC) pursuant to Section 401 of the Clean Water Act.

The proposed change would permit substantial simplification of Section 4.1.1.a without changing the scope of the survey program as set forth in the ETS, resulting in elimination of duplicative regulation. Furthermore, this change would eliminate the need for future ETS changes to conform with any 401 Certification changes which might be made by DEC. You will, in accordance with the revised ETS, continue to provide us with all reports submitted to DEC concerning the thermal plume mapping program.

We find that your proposal to consolidate requirements for thermal plume mapping in one section of the ETS and to refer to the 401 Certification to be acceptable. The changes involved here are procedural treatment of non-safety related environmental monitoring programs.

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Therefore, we have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Concl'd

OFFICE➤					
SURNAME➤					
DATE➤					

Since the amendment applies only to procedural treatment of environmental monitoring programs, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
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ACRS (16)
TERA
J. Buchanan
EB, PSB, EEB, RSB
W. Russell

Why do we refer to Unit 1 separately in the Notice, since there is only one site Tech Spec?

TAC #11621

#6 20A/17-B1

OFFICE	DOR:ORB1	DOR:ORB1		DOR:AD:ORP	OELD	DOR:ORB1
SURNAME	VLOlshan:jb	CSParrish		WPGammill	S.H. Lewis	ASchwencer
DATE	06/27/79	6/29/79		7/ /79	7/6/79	7/11/79

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 11, 1979

Docket No. 50-286

Mr. George T. Berry, Executive Director
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 12, 1979.

The amendment revises the Environmental Technical Specifications (ETS) to consolidate all references to thermal plume mapping into Section 4.1.1.a, which, in turn, refers to the conditions of the New York State Certification issued by the Department of Environmental Conservation (DEC) pursuant to Section 401 of the Clean Water Act.

The proposed change would permit substantial simplification of Section 4.1.1.a without changing the scope of the survey program as set forth in the ETS, resulting in elimination of duplicative regulation. Furthermore, this change would eliminate the need for future ETS changes to conform with any 401 Certification changes which might be made by DEC. You will, in accordance with the revised ETS, continue to provide us with all reports submitted to DEC concerning the thermal plume mapping program.

We find that your proposal to consolidate requirements for thermal plume mapping in one section of the ETS and to refer to the 401 Certification to be acceptable. The changes involved here are procedural treatment of non-safety related environmental monitoring programs.

Therefore, we have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to procedural treatment of environmental monitoring programs, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 27 to DPR-64
2. Notice of Issuance

cc: w/enclosures
See next page

cc: White Plains Public Library
100 Martine Avenue
White Plains, New York 10601

Mr. Vito J. Cassan
Assistant General Counsel
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Anthony Z. Roisman
Natural Resources Defense Council
917 - 15th Street, N.W.
Washington, D. C. 20005

Dr. Lawrence R. Quarles
Apartment 51
Kendal at Longwood
Kennett Square, Pennsylvania 19348

Mr. George M. Wilverding
Licensing Supervisor
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Mr. P. W. Lyon
Manager - Nuclear Operations
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Director, Technical Development
Programs
State Of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Attorney General
Department of Law
State Capitol
Albany, New York 12224

Mr. J. P. Bayne, Resident Manager
Indian Point 3 Nuclear Power Plant
P. O. Box 215
Buchanan, New York 10511

Mr. J. W. Blake, Ph.D., Director
Environmental Programs
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Theodore A. Rebelowski
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

Edward J. Sack, Esquire
Law Department
Consolidated Edison Company of
New York Inc.
4 Irving Place
New York, New York 10003

Jeffrey C. Cohen, Esquire
New York State Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223

Honorable George Begany
Mayor, Village of Buchanan
188 Westchester Avenue
Buchanan, New York 10511

cc: Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 12, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 27, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 11, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Replace the following pages of the Appendix "B" Environmental Technical Specifications with the enclosed pages. The revised pages are identified by amendment numbers in the lower left corner. Since these pages are common to the Technical Specifications in all the Indian Point Units, the amendment numbers applicable to each unit are listed on each page. Vertical lines in the right margin indicate the area of change associated with this amendment action.

Remove Pages	Insert Pages
2.1-1	2.1-1
2.1-2	2.1-2
2.1-3	2.1-3
2.1-4	2.1-4
2.1-6	2.1-6
2.1-8	2.1-8
2.1-9	2.1-9
2.2-5	2.2-5
4.1-1	4.1-1
4.1-2	
4.1-3	
4.1-4	
4.1-5	
4.1-6	
4.1-7	
5.6	5.6

2.0 LIMITING CONDITIONS FOR OPERATION

General: During a national power emergency, regional emergency, reactor emergency or an emergency need for power the limiting conditions for operation (LCO) provided in these Environmental Technical Specifications shall be inapplicable. During such emergencies, however, the LCO shall not be exceeded except as is necessitated by the emergency.

Applicability

Applies to the controlled release of thermal discharges, total residual chlorine and other chemical discharges, radioactive liquid, gaseous waste effluents and solid waste from the Indian Point Station.

Objective

To define the conditions for controlled release of nonradioactive and radioactive liquids and nonradioactive solids to the Hudson River and nonradioactive and radioactive gases to the atmosphere in order to assure compliance with applicable Federal and State regulations and to limit the stress to the aquatic ecosystem that might be caused by the discharge of excess concentrations or heat.

2.1 THERMAL

Applicability

Applies to the discharge of the heated water from the discharge structure.

3.0 MONITORING REQUIREMENTS

Applicability

Applies to routine sampling and analysis of the Station effluents and to an analytical evaluation of the data collected from the environmental monitoring survey.

Objective

To establish a sampling and analysis program which will assure that all effluents are kept within applicable Federal and State regulations.

3.1 THERMAL

Applicability

Applies to temperature measurements made in the intake forebays, and inside the discharge canal. Amendment No. 26, Unit 1
Amendment No. 57, Unit 2
Amendment No. 27, Unit 3

2.0 LIMITING CONDITIONS FOR OPERATION

Objective

To define the conditions for discharge of the heated water to assure compliance with applicable Federal and State regulations and to limit stresses to the aquatic ecosystem.

2.1.1 Maximum ΔT_c Across Circulating Water System (CWS)

Objective

To limit the maximum temperature rise across the CWS during full and reduced flow at all power levels.

Specification

2.1.1.1 The maximum temperature rise across the CWS shall not exceed the following temperature differentials, subject to the conditions of Section 2.1.2:

(a) 17F° when the CWS is operating at full flow and normal operation, or

(b) 28F° when the CWS is operating at reduced flow, or

3.0 MONITORING REQUIREMENTS

Objective

A. To assure that thermal protection conditions, including the temperature difference across the circulating water system, discharge temperature, and rate of temperature change, are maintained within the Environmental Technical Specifications.

3.1.1 Maximum ΔT_c Across Circulating Water System (CWS)

Objective

To monitor the intake and discharge temperatures at all power levels to assure that the allowable ΔT_c across the CWS is not exceeded.

Specification

3.1.1.1 The individual intake water temperatures along with the discharge canal water temperature shall both be continuously monitored and recorded. The intake water temperature shall be measured at a depth representative of the average intake temperature in a forebay of an intake for each Unit. The average temperature of the individual intake temperatures shall be calculated.

Amendment No. 26, Unit 1
Amendment No. 57, Unit 2
Amendment No. 27, Unit 3

2.0 LIMITING CONDITIONS FOR OPERATION

2.1.1.1 Specification (Cont'd)

(c) 31F° when the CWS is operating at reduced flow and one or more of the circulating water pumps is down for maintenance or pump outage, or

(d) 31F° when the CWS is operating at reduced flow and 10% of the normal condenser discharge is recirculated to the inlet for deicing purposes, or

(e) 38F° when the CWS is operating at reduced flow and 20% of the normal condenser discharge is recirculated to the inlet for deicing purposes.

(f) Item (a) above becomes 19F° when heater or preheater drains empty directly into condenser or during periods of high air inleakage into condenser.

(g) The limits specified in this Section may be exceeded if one or more of the circulating water pumps are down for maintenance or pump outage.

2.1.1.2 Whenever the temperature increment (ΔT_c) is above the specified limits in Section 2.1.1.1 for more than 6 hours, an investigation shall be undertaken to determine the cause for the temperature increase and corrective action shall be taken to reduce the ΔT_c to within the specified limits within 24 hours. These occurrences with corrective action shall be recorded and reported in accordance with Section 5.6.2.1.b.

3.0 MONITORING REQUIREMENTS

3.1.1.2 The discharge canal water temperature shall be obtained by means of a probe located in the discharge canal before the confluence with the river, at a depth of 5.5 feet below mean low water (see Figure 2.1-1 for location of probes).

3.1.1.3 Methods of measurement shall include temperature sensors (RTDs) with an accuracy of ±0.5°F and a sensitivity of 0.1°F. Temperature data from permanent RTDs shall be readout on a recorder in the control room. A three way switch shall be used to select the individual intake temperatures for recording purposes. The difference in output of the sensor at the discharge canal from the average of the individual intake temperatures, i.e., the ΔT_c across the CWS, shall be recorded on a daily basis and reported in accordance with Section 5.6.1.1, Annual Environmental Operating Report. A strip chart record shall be kept for examination and the daily minimum, maximum and average values of the ΔT_c shall be reported.

3.1.1.4 Continuous temperature monitoring is required following changes in power level, during reduced flow and deicing operations until the ΔT_c across the CWS is stable within 5% of the expected value. Monitoring during all flow conditions shall be carried out by continuously measuring and recording information as to the heat load, the intake and discharge canal water temperatures, and the maximum ΔT_c across the CWS.

Amendment No. 26 , Unit 1
Amendment No. 57 , Unit 2
Amendment No. 27 , Unit 3

Specifications (Cont'd)

Bases

The rise of intake water temperature across the condenser is a fixed value based upon initial condenser design, condenser cleanliness,

3.1.1.4 Specification (Cont'd)

The flow rate through the CWS shall be logged each day and any changes recorded at the time of the change. The operating conditions measurements, flow rates, and results are to be reported in accordance with Section 5.6.1.1, Annual Environmental Operating Report.

3.1.1.5 The continuous temperature recorder shall not be inoperative for a period exceeding 14 days. As an alternative during the monitoring or recording system downtime for calibration or repairs, manual temperature readings in the intake structure and in the discharge canal in the location 2 in Figure 2.1-1 shall be obtained and recorded once during each shift.

Temporary malfunction of temperature monitoring systems shall not be restrictive on plant operations, providing manual temperature measurements are taken.

3.1.1.6 There shall be an annual channel calibration of the sensor systems and a monthly channel functional test of the sensor systems.

3.1.1.7 Deviations from this monitoring program shall be promptly reported in accordance with Section 5.6.2.1.a.

Bases

Temperature monitoring sensors (RTDs) in the forebay of the intake and inside the discharge canal at the confluence with the Hudson River

Bases (Cont'd)

the intake water temperature is less than 40°F, the flow will be reduced from 100% flow to 60% flow for Units Nos. 1 and 2, thereby resulting in an increase in the temperature differential across the CWS. This flow reduction was required by the New York State Department of Environmental Conservation in an agreement made with Consolidated Edison on April 28, 1972, to rescind an Order by the State on February 29, 1972, to shut down the circulating water pumps after a fish kill occurred at the intake structure during testing of the pumps for Unit No. 2. The purpose of flow reduction is to reduce the volume of river water used and the intake velocity through the outer fixed screens to reduce fish impingement during wintertime. Flow at Unit No. 3 will also be reduced to minimize impingement effects when the intake river water temperature is less than 40°F as required in Specification 2.2.1.2. Flow may also be reduced when the intake water temperatures are greater than 40°F, provided the temperature conditions of the New York State thermal criteria are maintained.

Since the temperature differential increases with reduced flow, maintenance requiring pump shutdown should be avoided during the summertime (when the ambient water temperature can reach 79°F) in order to avoid excessive thermal stresses on aquatic biota.

During the winter, warmed water from the discharge canal can be recirculated to the intake forebay of Unit No. 3 for deicing purposes to melt

Bases (Cont'd)

During reduced flow when the intake water temperature is greater than 40°F, an increased temperature differential across the CWS will result in increasing the surface temperature of thermal discharges. Besides the thermal effects on biota as they pass through the condensers, organisms which are in the river water will also be exposed to the thermal plume, and during reduced pump capacities at the same heat load, they will be exposed to higher temperatures than otherwise During periods when the ambient river water temperatures reaches about 79°F, many organisms will be living near their upper critical temperatures and probably above their thermal range of metabolic insensitivity. Thus to avoid changes in species composition or the biotic community, operation during reduced pump capacities, when the ΔT_c can be 28F°, will be limited by the maximum discharge canal water temperature of 98°F. Therefore operation with reduced flow is limited.

Amendment No. 26, Unit 1
Amendment No. 57, Unit 2
Amendment No. 27, Unit 3

2.0 LIMITING CONDITIONS FOR OPERATION

Specification (Cont'd)

2.1.2.2 If this temperature is exceeded for two (2) hours, corrective action shall be taken to restore compliance with specifications unless there exists an emergency need for power.

2.1.2.3 When the discharge canal head differential is less than 1.5 feet for more than two (2) hours, the maximum temperature in the discharge canal shall not exceed 90°F. See Specification 2.2.2.2.

2.1.2.4 Deviation from the specifications for more than 24 hours shall be promptly reported in accordance with Section 5.6.2.1.a.

Bases

The analysis is based on the assumption that the maximum ambient temperature is 79°F and the maximum effect of recirculation on the intake temperature, from the tidal influence of the estuary, is a temperature differential

3.0 MONITORING REQUIREMENTS

Specification (Cont'd)

3.1.2.2 Temperatures in the discharge canal will be transmitted to the same control room as that of the intake and daily maximum, minimum and average temperatures reported in accordance with Section 5.6.1.1 Annual Environmental Operating Report.

3.1.2.3 The temperature measurements shall be visually displayed for monitoring purposes, recorded and reported in accordance with Section 5.6.1.1, Annual Environmental Operating Report.

3.1.2.4 When the above monitoring system is not operative, an alternative backup system as presented in Section 3.1.1.5 shall be used.

3.1.2.5 Deviations from the monitoring program shall be promptly reported in accordance with Section 5.6.2.1.a.

Bases

The placement of the temperature monitoring instrument in the discharge canal will give the temperature of the discharge water immediately before mixing with the receiving

Bases (Cont'd)

of less than 1.2F°. An upper limit of 98°F of the discharge water at the discharge port is thereby being set.

The thermal discharges shall also be maintained at all times to adequately protect aquatic biota against exposure to excess temperatures and to comply with the New York State thermal criteria.

2.1.3 Maximum Heat Rejection Rate (Btu/hr)

Objective

To limit the maximum heat discharged with the heated coolant water into the Hudson River.

Specification

2.1.3.1 The maximum rate of heat rejected into the river with the discharged heated coolant water, determined on a daily average basis, shall not exceed the sum of 16.3×10^9 Btu/hr.

All thermal discharges shall occur through the subsurface ports of the outfall structure.

Bases

The heat rejected is a function of the reactor core design, the primary

Bases (Cont'd)

water. The placement of this temperature sensor at a 5.5 foot depth in the discharge canal will provide for temperature measurements representative of the discharge water before mixing with the receiving water.

3.1.3 Maximum Heat Rejection Rate (Btu/hr)

Objective

To calculate the maximum heat rejection rate.

Specification

3.1.3.1 The heat output in Btu/hr for all Units in operation shall be averaged daily and reported in accordance with Section 5.6.1. Annual Environmental Operating Report.

Bases

Calculation of the heat rejection rate to the river will ba made for use in determining compliance with the limiting condition of operation contained in Section 2.1.3.

2.0 LIMITING CONDITIONS FOR OPERATION

Specification (Cont'd)

2.2.2.2 At or below 50% of the sum of the rated power levels of the three Units, a discharge velocity through the discharge ports shall be maintained such that the thermal discharges shall satisfy the New York State thermal criteria. When the discharge canal head differential is less than 1.5 feet, the temperature in the discharge canal shall not exceed 90°F.

2.2.2.3 If the head differential is not maintained at the required level beyond 24 hours, such deviation shall be promptly reported in accordance with Section 5.6.2.1.a.

3.0 MONITORING REQUIREMENTS

Specification (Cont'd)

3.2.2.2 The relationship between discharge velocity, open port area, and canal head above river level shall be confirmed by actual measurement and reported in accordance with Section 5.6.1.1.

3.2.2.3 Results of all adjustments to the gates of each of the port holes of the discharge structure and optimum mode of operation of the CWS through the intake-discharge structure, including the above mentioned relationship shall be recorded and reported in accordance with Section 5.6.1.1. Results of the discharge velocity measurements shall also be recorded and reported in accordance with Section 5.6.1.1, Annual Environmental Operating Report.

3.2.2.4 Deviations from these specifications beyond 24 hours shall be promptly reported in accordance with Section 5.6.2.1.a.

4.1 Nonradiological Environmental Surveillance

4.1.1.a Thermal Plume Mapping

The requirements of this thermal plume mapping program are contained under Conditions III.C.3 and III.C.4 of the New York State Department of Environmental Conservation ("DEC") 401 Certification issued May 2, 1975, and as may be amended by the DEC. Actions required based on the progress of the program are those contained in the 401 Certification. The objectives of the program, types of surveys, schedule for surveys and report requirements are based on those conditions. Those conditions are subject to interpretation and amendment by the DEC.

4.1.1.a.1 Specification

The licensees shall submit to the NRC, concurrently with submittal to the DEC , all reports required under conditions III.C.3 and III.C.4 of the 401 Certification. These submittals will satisfy the reporting requirements of Section 5.6.1.2 (Indian Point 1 and 2) and 5.6.1b (Indian Point 3).

Amendment NO. 26 , Unit 1
Amendment No. 57 , Unit 2
Amendment No. 27 , Unit 3

Results of all radiological environmental samples taken shall be summarized and tabulated on an annual basis. In the event that some results are not available within the 120 day period, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

5.6.1 b. Semiannual, and/or Special Environmental Operating Reports

(1) Non-Radiological

A Progress report and/or Annual Report shall be submitted by the licensee to the Director of Office of Nuclear Reactor Regulation by the end of July and the end of January, or as otherwise specified below, describing activities of the Thermal Plume Mapping and Ecological Survey Program, Entrainment Studies, Impingement Studies, and Special studies for the prior six-month interval. Information to be presented will include the following:

NOTE: These programs and studies will be performed in conjunction with Consolidated Edison, operators of Indian Point Units No. 1 and 2, and a joint report issued.

(a) Effects of chlorine and other chemical discharges on the ecosystem of the Hudson River in accordance with Sections 2.3 and 3.3 and 4.1.2a(2).

(b) Reduction in frequency of chlorination and reduction in concentration of free and combined residual chlorine in the discharge canal.

(c) Thermal plume model verification and mapping (near and far field) in accordance with Section 4.1.1.a.1.

(d) Ecological effects of thermal discharges in accordance with Section 4.1.2a(2).

(e) Potential reduction in dissolved oxygen in the cooling water through the plant.

(f) An assessment of performance of fish pumps as installed.

(g) Results of the general ecological survey in accordance with Section 4.1.2a(1).

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-3, 50-247, AND 50-386

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

POWER AUTHORITY OF THE STATE OF NEW YORK

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 26 and 57 to Provisional Operating License No. DPR-5 and Facility Operating License No. DPR-26, respectively, issued to Consolidated Edison Company of New York, Inc. and Amendment No. 27 to Facility Operating License No. DPR-64 issued to Power Authority of the State of New York (the licensees), which revised Technical Specifications for operation of the Indian Point Station, Unit No. 1 and Indian Point Nuclear Generating Unit Nos. 2 and 3 (the facilities) located in Buchanan, Westchester County, New York. These amendments are effective as of the date of issuance.

These amendments revised the Technical Specifications to consolidate all references to thermal plume mapping into Section 4.1.1.a which, in turn, refers to the conditions of the New York State Certification issued by the Department of Environmental Conservation pursuant to Section 401 of the Clean Water Act.

The application for amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made

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appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendment dated April 12, 1979; (2) Amendment Nos. 26 , 57 , and 27 to DPR-5, DPR-26, and DPR-64, respectively; and (3) the Commission's letter dated July 11, 1979.
All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 11th day of July, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors