

July 29, 1998

Mr. Paul H. Kinkel  
Vice President, Nuclear Power  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, NY 10511

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING UNIT  
NO. 2 (TAC NO. M98935)

Dear Mr. Kinkel:

The Commission has issued the enclosed Amendment No. 197 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated June 6, 1997, as supplemented by letter dated September 25, 1997.

The amendment revises TS Table 4.1-2, Frequency for Sampling Tests, to delete the requirement to sample the spray additive tank and delete the requirement for a sodium hydroxide (NaOH) spray additive in TS section 5.2.C.1.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed by:

Jefferey F. Harold, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 197 to DPR-26  
2. Safety Evaluation

cc w/encls: See next page

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NAME	JHarold: <i>efh</i>		JCushing <i>xc</i>	SLittle <i>sl</i>	SBajwa <i>sb</i>	MYoung
DATE	07/29/98		07/29/98	07/29/98	07/29/98	07/16/98

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PDR ADOCK 05000247  
P PDR

DATED: July 29, 1998

AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-26-INDIAN POINT UNIT 2

Docket File  
PUBLIC  
PDI-1 Reading  
J. Zwolinski (A)  
S. Bajwa  
S. Little  
J. Harold  
J. Cushing  
OGC  
G. Hill (2), T-5 C3  
W. Beckner, 013/H15  
ACRS  
J. Rogge, Region I

cc: Plant Service list

DATED: July 29, 1998

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Sincerely,

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Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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OFFICE	PM:PDI-1	E	PM:PDI-1	LA:PDI-1	D:PDI-1	OGC*
NAME	JHarold: lco/ll		JCushing	SLittle	SBajwa	MYoung
DATE	07/29/98		07/29/98	07/29/98	07/29/98	07/16/98

Official Record Copy



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 29, 1998

Mr. Paul H. Kinkel  
Vice President, Nuclear Power  
Consolidated Edison Company  
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Broadway and Bleakley Avenue  
Buchanan, NY 10511

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Jeffrey F. Harold".

Jefferey F. Harold, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 197 to DPR-26  
2. Safety Evaluation

cc w/encls: See next page

Paul H. Kinkel  
Consolidated Edison Company  
of New York, Inc.

Indian Point Nuclear Generating  
Station Units 1/2

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 6, 1997, as supplemented by letter dated September 25, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.197 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 29, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 197

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

5.2-2

Table 4.1-2 page 1 of 2

Insert Pages

5.2-2

Table 4.1-2 page 1 of 2

C. CONTAINMENT SYSTEMS

1. The containment vessel has an internal spray system which is capable of providing a distributed borated water spray of at least 2200 gpm<sup>(3)</sup>.
2. The containment vessel has an internal air recirculation system which includes five fan-cooler units (centrifugal fans and water cooled heat exchangers), with a total heat removal capability of at least 308.5 MBtu/hr under conditions following a loss-of-coolant accident and at service water temperature of 95°F.<sup>(4)</sup> All of the fan cooler units are equipped with activated charcoal filters to remove volatile iodine following an accident.

References

- (1) UFSAR Section 5.1.2.2
- (2) UFSAR Section 5.1.4
- (3) UFSAR Section 6.3
- (4) UFSAR Section 6.4

Table 4.1-2

Frequencies for Sampling Tests

	Check	Frequency	Maximum Time Between Tests	
1.	Reactor Coolant Samples	Gross Activity (1) Radiochemical(2) E Determination Tritium Activity F, Cl & O <sub>2</sub>	5 days/week (1) Monthly Semi-annually (3) Weekly (1) Weekly	3 days 45 days 30 weeks 10 days 10 days
2.	Reactor Coolant Boron	Boron Concentration	Twice/week	5 days
3.	Refueling Water Storage Tank Water Sample	Boron Concentration	Monthly	45 days
4.	Boric Acid Tank	Boron Concentration	Twice/week	5 days
5.	DELETED			
6.	DELETED			
7.	Accumulator	Boron Concentration	Monthly	45 days
8.	Spent Fuel Pit	Boron Concentration	Monthly	45 days
9.	Secondary Coolant	Iodine-131	Weekly (4)	10 days
10.	Containment Iodine- Particulate Monitor or Gas Monitor	Iodine-131 and Particulate Activity or Gross Gaseous Activity	Continuous When Above Cold Shutdown(5)	NA*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO.197 TO FACILITY OPERATING LICENSE NO. DPR-26  
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2  
DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated June 6, 1997, as supplemented by letter dated September 25, 1997, the Consolidated Edison Company of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications (TSs). The requested changes would revise TS Table 4.1-2, Frequency for Sampling Tests, to delete the requirement to sample the spray additive tank and delete the requirement for a sodium hydroxide (NaOH) spray additive in TS Section 5.2.C.1. The September 25, 1997, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

On April 23, 1997, the Commission reviewed and approved Amendment No. 191 to Operating License No. DPR-26. Amendment No. 191 eliminated the requirement for the NaOH containment spray additive and the spray additive tank. Consolidated Edison's request inadvertently failed to include the deletion of the requirement to sample the spray additive tank and the reference to NaOH in TS 5.2.C.1 Containment Systems.

The removal of the spray additive tank was analyzed and approved by the Commission in Amendment No. 191 to Operating License No. DPR-26. The analysis that supported the removal of the tank also supports removing the requirement to sample the tank. The ph control will be provided by trisodium phosphate stored in baskets located in the containment sump. NaOH is no longer used for ph control. The staff finds it acceptable to delete the requirement to sample the spray additive tank.

Removing the requirement for a NaOH containment spray additive in TS 5.2.C.1, Containment Systems, is also supported by Amendment No. 191 to Operating License No. DPR-26. NaOH is no longer supplied to the spray water for ph control and iodine removal; therefore, this reference is no longer correct. Trisodium phosphate stored in baskets located in the containment sump is now used for ph control. The staff finds it acceptable to delete the requirement for sodium hydroxide spray additive used for iodine removal referenced in TS Section 5.2.C.1.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (63 FR 4310). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Cushing

Date: July 29, 1998