

Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616-465-5901



December 12, 2000

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-58 and DPR-74
Docket Nos. 50-315 and 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

LER 50-315/2000-008-00, "Failure To Test Essential Service Water Valves In Accordance With Technical Specification Surveillance Requirements"

The cause of this event is presently under investigation and will be reported in a supplement to this report.

This report is being submitted one day late because of incimate weather. This was discussed with Mr. A. Vogel of your staff on December 11, 2000.

Should you have any questions regarding this correspondence, please contact Mr. Wayne J. Kropp, Director Regulatory Affairs, at 616/697-5056.

Sincerely,

A handwritten signature in black ink that reads 'Joseph E. Pollock'. The signature is written in a cursive style with a large, looped initial 'J'.

Joseph E. Pollock
Plant Manager

/rmc
Attachment

c: J. E. Dyer, Region III
D. Hahn
B. A. McIntyre
T. P. Noonan
A. C. Bakken III
R. P. Powers
R. Whale
NRC Resident Inspector
Records Center, INPO

IE22

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1) Donald C. Cook Nuclear Plant Unit 1		DOCKET NUMBER (2) 05000-315	PAGE (3) 1 of 1
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TITLE (4)
Failure to Test Essential Service Water Valves In Accordance With Technical Specification Surveillance Requirements

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
3	16	2000	2000	-- 008 --	00	12	12	2000	Cook Unit 2	05000-316	
									FACILITY NAME	DOCKET NUMBER	

OPERATING MODE (9) ---	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)										
	20.2201 (b)			20.2203(a)(2)(v)			<input checked="" type="checkbox"/>		50.73(a)(2)(i)		50.73(a)(2)(viii)
POWER LEVEL (10) 0	20.2203(a)(1)			20.2203(a)(3)(i)					50.73(a)(2)(ii)		50.73(a)(2)(x)
	20.2203(a)(2)(i)			20.2203(a)(3)(ii)					50.73(a)(2)(iii)		73.71
	20.2203(a)(2)(ii)			20.2203(a)(4)					50.73(a)(2)(iv)		OTHER
	20.2203(a)(2)(iii)			50.36(c)(1)					50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A
20.2203(a)(2)(iv)			50.36(c)(2)					50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)											
NAME R. W. Gaston, Regulatory Affairs								TELEPHONE NUMBER (Include Area Code) 616 / 465-5901, x1366			

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)			MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	YES (If Yes, complete EXPECTED SUBMISSION DATE).					<input type="checkbox"/>	NO		2	26	2001

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)
 On March 16, 2000, during development of the new Essential Service Water (ESW) flow balance procedure to replace the old procedure, it was discovered that the Component Cooling Water (CCW) Heat Exchanger Essential Service Water (ESW) outlet valves for Unit 1 (1-WMO-733 and 1-WMO-737) and Unit 2 (2-WMO-734 and 2-WMO-738) have not been fully tested in accordance with Technical Specification 3/4.7.4.1.b. Specifically, on a refueling outage basis the valves are required to be demonstrated to actuate to their correct position on a Safety Injection (SI) signal. However, the previous procedure only required the valves to be demonstrated to reposition to their intermediate position from the fully open position. In that these valves may be closed during normal operation when the opposite CCW train is the only train in service, repositioning to the intermediate position from the fully closed position on a SI signal is a design basis scenario and should have also been demonstrated.

On November 9, 2000, it was determined that the initial reportability determination was in error. Upon re-review, it was determined that this condition was reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

The cause of this event is presently under investigation and will be reported in a supplement to this report.

The Unit 2 ESW valves were properly tested on March 31, 2000.
 The Unit 1 ESW valves were properly tested on November 01, 2000.