July 19, 1989

Docket No. 50-247

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Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 71776)

The Commission has issued the enclosed Amendment No. 142 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 28, 1988, as clarified May 10, 1989.

The amendment adds a new surveillance requirement to Technical Specification 4.16 requiring periodic leak testing of the low head injection line check valves 897A-D and RHR check valves 838A-D. The amendment also makes certain editorial changes in Technical Specification 4.16 which are administrative in nature.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:
1. Amendment No. 142 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

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[AMEND 247 TAC 71776]

OFC	PDI-1	PDI-1	OGC	PDI-1	OGC		
NAME	CVogan	DBrinkman/vr/bah		RCapra	APH		
DATE	7/7/89	7/7/89	/ /89	7/18/89	7/12/89		

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C/P-1

Mr. Stephen B. Bram
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated December 28, 1988, as clarified May 10, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.142 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective no later than 30 days after its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 19, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages	Insert Pages
4.16-1	4.16-1
4.16-2	4.16-2

4.16 REACTOR COOLANT SYSTEM AND CONTAINMENT FREE VOLUME LEAKAGE DETECTION AND REMOVAL SYSTEMS SURVEILLANCE

Applicability

Applies to the surveillance and monitoring of leakage detection and removal systems provided for determining and removing reactor coolant leakage and leakage into the containment free volume. Applies to the testing of certain LPI/RHR check valves(1,2).

Objective

To verify compliance with operational leakage limits of Specification 3.1.F. To specify a test to check for RCS leakage through certain check valves.

Specifications

A. For the purposes of demonstrating compliance with the operational leakage limits of Specification 3.1.F., the following shall be performed:

1. At least once a shift, monitor the leakage detection systems required by Specification 3.1.F.1.a(6).

2. At least once a shift, monitor the containment sump inventory and discharge.

3. At least once a shift, monitor the recirculation sump inventory and the reactor cavity inventory.

4. At least once daily, perform a reactor coolant system water inventory balance.

5. For the RCS/RHR pressure isolation valves, periodic leakage testing* shall be accomplished every time the plant is placed

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* To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria. Minimum test differential pressure shall not be less than 150 psid.

in the cold shutdown condition for refueling, each time the plant is placed in a cold shutdown condition for at least 72 consecutive hours if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair or replacement work is performed.

B. A test shall be performed, whenever the RCS pressure decreases to 700 psig (i.e. within 100 psig of the RHR design pressure) or whenever the RHR is secured to go to hot shutdown, to check for leakage through SIS low head injection line check valves 897A-D and RHR check valves 838A-D.

C. The containment sump pumps required to be operable by Specification 3.1.F.1.a(1) shall be demonstrated to be operable by performance of the following surveillance program:

1. At monthly intervals, each sump pump shall be started and a discharge flow of at least 25 gpm verified.

2. At refueling intervals, each sump pump shall be operated under visual observation to verify that the pumps start and stop at the appropriate setpoints and that the discharge flow is at least 25 gpm per pump.

Basis

Specifications 4.16.A and 4.16.C establish the surveillance program for monitoring reactor coolant system leakage and leakage into the containment free volume during plant operation and ensure compliance with Specification 3.1.F. These specifications also establish surveillance requirements for the containment sump pumps. Surveillance requirements for the various leakage detection instrumentation systems are contained in Table 4.1-1 of these specifications.

Specification 4.16.B was added to the Technical Specifications in response to NRC's July 5, 1985 rescission of our February 11, 1980 Confirmatory Order Item A.5. Item A.5 was developed to address the intersystem loss-of-coolant accident (Event V) identified in WASH-1400[1,2]. The RHR system design pressure is 600 psig.

References

(1) NRC Letters dated July 5, 1985, and February 11, 1980
(2) Con Edison Letter dated March 14, 1980

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

INTRODUCTION

By letter dated December 28, 1988, as clarified May 10, 1989, the Consolidated Edison Company of New York, Inc. (the licensee) requested that a new surveillance test requirement be added to Technical Specification (TS) 4.16 to require periodic leak testing of low head injection line check valves 897A-D and residual heat removal (RHR) check valves 838A-D. The following evaluation addresses the licensee's proposed TS change.

EVALUATION

The licensee proposed adding a new TS which would require periodic leak testing of low head injection line check valves 897A-D and RHR check valves 838A-D. The new TS would replace the licensee's commitments to leak test these valves as stated in the licensee's February 1, 1980 letter. The licensee's commitments were confirmed by Item A.5 in the NRC's Confirmatory Order dated February 11, 1980. Item A.5 of the Order requires the licensee to conduct testing to assure that the subject check valves are in fact installed correctly and functioning as pressure isolation barriers when the plant is at power. The Order requires verification of check valve operability whenever RCS pressure has decreased to within 100 psig of RHR system pressure.

On July 15, 1985, the NRC issued a Rescission of Order of the February 11, 1980 Confirmatory Order. However, the Rescission did not immediately rescind Item A.5 of the Confirmatory Order but rather required the licensee to follow Item A.5 of the Order until Technical Specifications implementing the requirements of a generic letter dated February 23, 1980, entitled "LWR Primary Coolant System Pressure Isolation Valves" are incorporated into the Indian Point Unit 2 license. The Rescission provided that upon approval of such TS, the Confirmatory Order would be rescinded with respect to Item A.5.

The licensee's March 14, 1980 response to the February 23, 1980 generic letter committed the licensee to test the subject check valves whenever the RCS

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pressure decreased to within 100 psig of the RHR system design pressure. The proposed new surveillance requirement to be added to TS 4.16 is consistent with the licensee's commitment to the February 23, 1980 generic letter and with the requirements of Item A.5 of the Confirmatory Order. Therefore, the staff finds the proposed TS acceptable. By adding this new surveillance requirement to TS 4.16, Item A.5 of the Confirmatory Order is considered rescinded.

The licensee also proposed certain editorial changes (e.g., delete underlines, add commas, move text to another page, etc.) in TS 4.16. The proposed changes are purely administrative in nature and are therefore acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20 and adds a surveillance requirement. The staff has determined that this amendment involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 19, 1989

PRINCIPAL CONTRIBUTOR:

D. S. Brinkman