

REGULATORY DOCKET FILE COPY

DOCKET FILE 50-247

Docket No. 50-247

August 28, 1980

Mr. William J. Cahill, Jr.
Vice President
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. **62** to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your applications transmitted by letters dated February 9, 1979, December 31, 1979, and June 23, 1980.

The amendment reduces the number of hydraulic snubbers required by the Technical Specifications, reflecting redesign of the support systems of certain safety-related pipe lines.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. **62** to DPR-26
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

8009100631

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

Joseph D. Neighloes
for Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 62 to DPR-26
2. Safety Evaluation
3. Notice of Issuance

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See next page

Mr. Peter Zarakas
Consolidated Edison Company of New York, Inc.

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August 28, 1980

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated February 9, 1979 (supplemented by letters dated December 31, 1979 and June 23, 1980), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Joseph D. Neighloes
for Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages
Table 3.12-1
(all 18 sheets)
4.12-1
4.12-2

Insert Pages
Table 3.12-1
(14 sheets)
4.12-1
4.12-2

TABLE 3.12 - 1 (SHEET 1 OF 14)

Safety Related Shock Suppressors (Snubbers)

Line No.	Snubber No.	Location (Approx.)	Category
1	MSR-2V	VC EL 101'-0"	3
1	SR-M4	AFB EL 62'-10"	4
1	SR-M5A	AFB EL 62'-10"	4
1	SR-M5B	AFB EL 62'-10"	4
2	SR-M2	AFB EL 75'-6"	4
2	SR-M3A	AFB EL 75'-6"	4
2	SR-M3B	AFB EL 75'-6"	4
2	SR-M1	AFB EL 75'-6"	4
2	SR-M50	AFB EL 75'-6"	4
2	SR-M51	AFB EL 75'-6"	4
3	MSR-1V	VC EL 101'-0"	3
3	SR-M6	AFB EL 75'-6"	4
3	SR-M7	AFB EL 75'-6"	4
3	SR-M8A (two)	AFB EL 75'-6"	4
3	SR-M8B	AFB EL 75'-6"	4
3	SR-M53	AFB EL 75'-6"	4
4	SR-M9	AFB EL 62'-10"	4
4	SR-M10A	AFB EL 62'-10"	4
4	SR-M10B	AFB EL 62'-10"	4
4	SR-M55	AFB EL 62'-10"	4
4	SR-M56	AFB EL 62'-10"	4
5	SR-B-3	AFB EL 39'-6"	4
5	SR-B-4**	AFB EL 39'-6"	4
5	SR-B-9**	AFB EL 35'-0"	4
6	BF-SR-9	VC EL 59'-6"	3
6	SR-B1	AFB EL 41'-0"	4

Amendment No. 62

TABLE 3.12 - 1 (SHEET 2 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
6	SR-B2**	AFB EL 40'-0"	4
7	SR-B7	AFB EL 40'-7"	4
7	SR-B8**	AFB EL 41'-0"	4
8	SR-B5	AFB EL 39'-6"	4
8	SR-B6**	AFB EL 39'-6"	4
8	SR-B10**	AFB EL 35'-0"	4
9	9-SR-1**	VC EL 59'-6"	3
9	SR-55	PAB EL 27'-0"	4
9	SR-801**	VC EL 56'-0"	3
9	SR-802**	VC EL 54'-7"	3
9	SR-803**	VC EL 54'-7"	3
9	SR-803A**	VC EL 54'-7"	3
9	SR-804**	VC EL 54'-3"	3
9	SR-805**	VC EL 54'-7"	3
10	SR-65	PAB EL 27'-0"	4
10	SR-807	VC EL 59'-6"	3
10	SR-807A**	VC EL 59'-6"	3
10	10-SR-807B**	VC EL 59'-6"	3
10	SR-807C**	VC EL 59'-6"	3
10	SR-808**	VC EL 59'-6"	3
10	SR-809	VC EL 59'-6"	3
10	SR-809A	VC EL 59'-6"	3
10	SR-810**	VC EL 59'-6"	3
10	SR-811**	VC EL 58'-2"	3
13	13-SR-1**	VC EL 58'-0"	3
13	SR-935	VC EL 69'-0"	3

Amendment No. 62

TABLE 3.12 - 1 (SHEET 3 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
13	SR-936	VC EL 76'-9"	3
13	SR-937	VC EL 84'-0"	3
13	SR-937A	VC EL 84'-0"	3
13	SR-938	VC EL 84'-9"	3
13	SR-939	VC EL 76'-2"	3
13	SR-1027A	VC EL 80'-6"	3
13	SR-1030	VC EL 68'-0"	3
13	SR-1030A	VC EL 68'-0"	3
13	SR-1031	VC EL 76'-0"	3
13	SR-1032	VC EL 76'-8"	3
13	SR-1037	VC EL 83'-7"	3
13	SR-1037A	VC EL 83'-7"	3
13	SR-1051	VC EL 84'-3"	3
13	SR-1052	VC EL 76'-9"	3
13	SR-1053	VC EL 68'-0"	3
13	SR-1060	VC EL 76'-0"	3
13	SR-1079	VC EL 68'-0"	3
13	SR-1080	VC EL 75'-0"	3
13	SR-1099	VC EL 82'-0"	3
13	SR-1100	VC EL 76'-0"	3
13	SR-1103	VC EL 76'-0"	3
13	SR-1104	VC EL 82'-0"	3
13	SR-1105	VC EL 65'-10"	3
13	13-SR-1105A **	VC EL 66'-6"	3
13	SR-1106	VC EL 65'-10"	3
13	SR-1124	VC EL 76'-6"	3

TABLE 3.12 - 1 (SHEET 4 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
14	14-SR-1	VC EL 83'-0"	3
14	SR-925	VC EL 84'-9"	3
14	SR-927	VC EL 81'-6"	3
14	SR-927A	VC EL 81'-6"	3
14	SR-928	VC EL 75'-9"	3
14	SR-928A	VC EL 76'-4"	3
14	SR-969	VC EL 76'-0"	3
14	SR-970	VC EL 75'-0"	3
14	SR-971	VC EL 74'-2"	3
14	SR-1035	VC EL 73'-0"	3
14	SR-1036A	VC EL 76'-0"	3
14	SR-1039A	VC EL 84'-0"	3
14	SR-1040A	VC EL 84'-0"	3
14	SR-1042	VC EL 75'-9"	3
14	SR-1049	VC EL 75'-8"	3
14	SR-1050	VC EL 84'-3"	3
14	SR-1057	VC EL 84'-3"	3
14	SR-1083	VC EL 78'-5"	3
14	SR-1084	VC EL 78'-0"	3
14	SR-1093	VC EL 76'-8"	3
14	SR-1094	VC EL 69'-0"	3
14	SR-1095	VC EL 76'-8"	3
14	SR-1096	VC EL 69'-0"	3
14A	SR-954	VC EL 66'-6"	3
14A	SR-955**	VC EL 71'-0"	3
14A	SR-955A**	VC EL 71'-0"	3

TABLE 3.12 - 1 (SHEET 5 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
14A	SR-1001	VC EL 68'-8"	3
14A	SR-1002	VC EL 69'-0"	3
14A	SR-1075	VC EL 70'-6"	3
14A	SR-1076	VC EL 66'-3"	3
14A	SR-1078	VC EL 70'-3"	3
14A	SR-1120	VC EL 68'-8"	3
14A	SR-1122	VC EL 69'-0"	3
14A	SR-1123	VC EL 68'-8"	3
14A	SR-1077	VC EL 70'-6"	3
16	56-SR-1	VC EL 61'-3"	3
17	17-SR-1	VC EL 58'-0"	3
17	17-SR-2	VC EL 58'-0"	3
17	17-SR-3	VC EL 58'-0"	3
17	17-SR-4	VC EL 58'-0"	3
17	SR-941	VC EL 75'-7"	3
17	SR-941A	VC EL 75'-7"	3
17	SR-1010	VC EL 76'-6"	3
17	SR-1069	VC EL 76'-0"	3
17	SR-1112	VC EL 68'-0"	3
17	SR-1113	VC EL 69'-0"	3
17	SR-1116	VC EL 65'-0"	3
17	SR-1117	VC EL 65'-0"	3
17	SR-1118	VC EL 69'-0"	3
19	SR-896	VC EL 57'-6"	3
27	SR-898**	VC EL 58'-3"	3
38	38-SR-11	VC EL 59'-3"	3

TABLE 3.12 - 1 (SHEET 6 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
38	38-SR-20	VC EL 55'-0"	3
38	38-SR-21	VC EL 60'-0"	3
38	38-SR-22	VC EL 60'-0"	3
38	38-SR-23	VC EL 60'-0"	3
38	38-SR-24	VC EL 60'-0"	3
41	SR-952	VC EL 76'-0"	3
41	SR-953	VC EL 68'-0"	3
41	SR-953A	VC EL 65'-0"	3
43	SR-1020A	VC EL 81'-0"	3
43	SR-1024A	VC EL 74'-5"	3
43	SR-1025A	VC EL 69'-11"	3
43	SR-1026	VC EL 68'-9"	3
44	SR-1072	VC EL 68'-3"	3
44	SR-1073	VC EL 68'-7"	3
45	45-SR-9	VC EL 65'-7"	3
45	45-SR-30	PAB EL 64'-0"	4
46	46-SR-2	VC EL 69'-0"	3
46	46-SR-3	VC EL 69'-0"	3
46	46-SR-30	PAB EL 64'-0"	4
47	47-SR-30	PAB EL 64'-0"	4
48	48-SR-30	PAB EL 64'-0"	4
56	56-SR-6	VC EL 55'-6"	3
56	56-SR-12	VC EL 63'-3"	3
56	56-SR-26	VC EL 50'-9"	3
60	SR-73A	PAB EL 71'-0"	4
60	SR-703 (two)	VC EL 55'-0"	3

TABLE 3.12 - 1 (SHEET 7 OF 14)

Safety Related Shock Suppressors (Snubbers)

Line No.	Snubber No.	Location (Approx.)	Category
60	SR-746A	VC EL 64'-6 $\frac{1}{2}$ "	3
60	SR-746B	VC EL 64'-6 $\frac{1}{2}$ "	3
60	SR-746C	VC EL 57'-0"	3
61	SR-881	VC EL 64'-2 $\frac{1}{2}$ "	3
61	SR-887	VC EL 68'-3"	3
62	SR-922A	VC EL 68'-0"	3
62	SR-922B	VC EL 68'-0"	3
62	SR-924	VC EL 70'-0"	3
70	70-SR-3	VC EL 85'-9 $\frac{1}{2}$ "	3
70	70-SR-4	VC EL 66'-10"	3
70	70-SR-5	VC EL 68'-6"	3
70	70-SR-6	VC EL 103'-0"	3
70	70-SR-10	VC EL 123'-9"	3
70	70-SR-11	VC EL 123'-9"	3
70	70-SR-12	VC EL 123'-9"	3
70	70-SR-13	VC EL 123'-9"	3
70	70-SR-14	VC EL 127'-3"	3
70	RCS-5	VC EL 102'-3 3/4"	3
70	RCS-6	VC EL 103'-0"	3
70	70-RCS-5A	VC EL 103'-0"	3
71	71-SR-1	VC EL 80'-0"	3
71	SR-963	VC EL 76'-0"	3
71	SR-964	VC EL 68'-0"	3
71	SR-964A	VC EL 68'-6"	3
71	SR-967A	VC EL 63'-10"	3
72	72-SR-1	VC EL 80'-0"	3

TABLE 3.12 - 1 (SHEET 8 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
72	SR-1125	VC EL 80'-0"	3
72	SR-1126	VC EL 70'-0"	3
72	SR-1127	VC EL 70'-0"	3
72	SR-1128	VC EL 72'-0"	3
72	SR-1129	VC EL 63'-0"	3
72	SR-1131	VC EL 64'-0"	3
73	SR-1016A	VC EL 76'-7"	3
73	SR-1017	VC EL 69'-1"	3
73	SR-1017A	VC EL 69'-0"	3
73	SR-1017B	VC EL 69'-0"	3
73	SR-1018A	VC EL 69'-0"	3
74	74-SR-1	VC EL 80'-4"	3
74	SR-1085	VC EL 67'-8"	3
74	SR-1086	VC EL 68'-9"	3
74	SR-1087	VC EL 68'-11"	3
74	SR-1087A	VC EL 70'-4"	3
74	SR-1089	VC EL 68'-9"	3
74	SR-1092	VC EL 71'-0"	3
76	76-H-15	VC EL 65'-0"	3
78	78-SR-1	VC EL 70'-6"	3
79	SR-901**	VC EL 51'-0"	3
79	SR-902	VC EL 56'-6"	3
79	SR-903**	VC EL 56'-6"	3
79	SR-904**	VC EL 56'-6"	3
79	SR-905**	VC EL 56'-6"	3
79	SR-906**	VC EL 56'-6"	3

TABLE 3.12 - 1 (SHEET 9 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
79	SR-907	VC EL 56'-6"	3
79	SR-908	VC EL 56'-6"	3
79	SR-909**	VC EL 56'-6"	3
79	SR-910**	VC EL 56'-6"	3
79	SR-911**	VC EL 46'-3"	3
79	SR-911A**	VC EL 55'-0"	3
80	SR-915B**	VC EL 58'-0"	3
80	SR-916**	VC EL 58'-0"	3
80	SR-920A	VC EL 58'-8"	3
80	SR-920B**	VC EL 58'-0"	3
93	SR-752	VC EL 73'-9"	3
93	SR-752A	VC EL 73'-9"	3
93	SR-753	VC EL 85'-3"	3
94	SR-759	VC EL 85'-3"	3
96	SR-913**	VC EL 58'-0"	3
155	SR-50A	PAB EL 25'-0"	4
163	SR-250	PAB EL 68'-6"	4
163	SR-250A	PAB EL 68'-6"	4
163	163-SR-5	PAB EL 48'-0"	4
250	250-SR-1	PAB EL 110'-0"	4
293	SR-761	VC EL 49'-2"	3
293	SR-763A	VC EL 50'-0"	3
317	317-SR-1	VC EL 65'-0"	3
317	SR-766	VC EL 57'-0"	3
317	SR-766A	VC EL 53'-6"	3
342	342-SR-6	VC EL 103'-0"	3

TABLE 3.12 - 1 (SHEET 10 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
343	343-SR-5	VC EL 103'-0"	3
344	344-SR-4	VC EL 103'-0"	3
351	PWR-127	VC EL 60'-0"	3
351	PWR-128	VC EL 55'-0"	3
351	PWR-129	VC EL 55'-0"	3
351	SR-742**	VC EL 56'-2 7/8"	3
351	351-SR-1	VC EL 55'-0"	3
352	PWR-152	VC EL 66'-6"	3
352	SR-713**	VC EL 51'-0"	3
352	352-SR-2**	VC EL 63'-0"	3
353	PWR-147A	VC EL 60'-0"	3
353	PWR-148	VC EL 48'-9"	3
353	SR-736	VC EL 47'-3 1/4"	3
353	SR-737	VC EL 48'-9"	3
353	SR-737A	VC EL 58'-6"	3
355	SR-748	VC EL 56'-0"	3
356	356-SR-1**	VC EL 55'-3"	3
356	SR-714**	VC EL 61'-6"	3
356	SR-715**	VC EL 61'-6"	3
356	SR-716	VC EL 61'-6"	3
356	SR-717**	VC EL 61'-6"	3
356	SR-718**	VC EL 61'-6"	3
356	SR-718A	VC EL 55'-3"	3
356	SR-720	VC EL 55'-3"	3
358	SR-738B	VC EL 55'-1 3/4"	3
358	SR-738A	VC EL 55'-4 1/8"	3

TABLE 3.12 - 1 (SHEET 11 OF 14)

Safety Related Shock Suppressors (Snubbers)

Line No.	Snubber No.	Location (Approx.)	Category
361	SR-724 **	VC EL 53'-6"	3
361	SR-726 **	VC EL 53'-6"	3
361	SR-728 **	VC EL 53'-6"	3
361	SR-729 **	VC EL 65'-3"	3
361	SR-730 **	VC EL 65'-3"	3
361	SR-732A	VC EL 65'-0"	3
361	SR-749	VC EL 53'-6"	3
361	SR-749A	VC EL 57'-0"	3
361	SR-749B	VC EL 55'-0"	3
361	SR-749C	VC EL 53'-6"	3
361	SR-755	VC EL 56'-9"	3
361	SR-756	VC EL 71'-6"	3
361	361-SR-10	VC EL 61'-6"	3
518	SR-71A	PAB EL 70'-0"	4
577	577-SR-1	VC EL 65'-1 1/2"	3
577	577-SR-5	VC EL 59'-6"	3
577	577-SR-13	VC EL 54'-11"	3
577	577-SR-15	VC EL 56'-6"	3
577	577-SR-17	VC EL 62'-0"	3
V-2	SR-V20A	AFB EL 55'-2"	4
V-2	SR-V20B	AFB EL 55'-2"	4
V-3	SR-M29	AFB EL 65'-10"	4
V-3	SR-M30	AFB EL 64'-0"	4
V-3	SR-M31	AFB EL 64'-0"	4
V-3	SR-M33	AFB EL 83'-6"	4
V-4	SR-M25	AFB EL 78'-6"	4
V-4	SR-M27	AFB EL 76'-0"	4

TABLE 3.12 - 1 (SHEET 12 OF 14)

Safety Related Shock Suppressors (Snubbers)

<u>Line No.</u>	<u>Snubber No.</u>	<u>Location (Approx.)</u>	<u>Category</u>
V-4	SR-M52	AFB EL 76'-0"	4
V-5	SR-M34	AFB EL 76'-3"	4
V-5	SR-M35	AFB EL 73'-6"	4
V-5	SR-M36	AFB EL 73'-6"	4
V-5	SR-M37	AFB EL 76'-3"	4
V-5	SR-M38**	AFB EL 85'-0"	4
V-5	SR-M54	AFB EL 76'-3"	4
V-6	SR-M39	AFB EL 64'-2"	4
V-6	SR-M40	AFB EL 64'-2"	4
V-6	SR-M41	AFB EL 64'-0 1/2"	4
MS-3	SR-M20A**	AFB EL 27'-5"	4
MS-3	SR-M21A**	AFB EL 20'-6"	4
MS-3	SR-M20B**	AFB EL 27'-5"	4
MS-3	SR-M21B**	AFB EL 20'-6"	4
MS-3	SR-M22A**	AFB EL 26'-7"	4
MS-3	SR-M22B**	AFB EL 26'-7"	4
MS-3	SR-M23B**	AFB EL 27'-6"	4
MS-3	SR-M24**	AFB EL 27'-6"	4
MS-3	SR-499	AFB EL 68'-0"	4
MS-3	SR-500**	AFB EL 68'-0"	4
MS-3	SR-501	AFB EL 68'-0"	4
MS-3	SR-501A**	AFB EL 65'-1"	4
MS-3	SR-501B**	AFB EL 65'-1"	4
MS-3	SR-502**	AFB EL 66'-0"	4
MS-3	SR-503	AFB EL 66'-8"	4
MS-3	SR-503A**	AFB EL 65'-11"	4

TABLE 3.12 - 1 (SHEET 13 OF 14)

Safety Related Shock Suppressors (Snubbers)

Line No.	Snubber No.	Location (Approx.)	Category
MS-3	SR-503B**	AFB EL 65'-1"	4
MS-3	SR-504**	AFB EL 58'-10"	4
MS-3	SR-505**	AFB EL 55'-10"	4
MS-3	SR-506**	AFB EL 51'-7"	4
MS-3	SR-507**	AFB EL 38'-6"	4
MS-3	SR-507A**	AFB EL 48'-5 1/8"	4
MS-3	SR-507B**	AFB EL 48'-7 5/8"	4
MS-3	SR-507C**	AFB EL 35'-5 7/8"	4
MS-3	MS-SR-129***	AFB EL 55'-10"	4
Steam Gen.			
#21	SG21-1 thru SG21-4	VC-E1.94'	2, 3
#21	SG21-5 and SG21-6	VC-E1.46'	2, 3
#22	SG22-1 thru SG22-4	VC-E1.94'	2, 3
#22	SG22-5 and SG22-6	VC-E1.46'	2, 3
#23	SG23-1 thru SG23-4	VC-E1.94'	2, 3
#23	SG23-5 and SG23-6	VC-E1.46'	2, 3
#24	SG24-1 thru SG24-4	VC-E1.94'	2, 3
#24	SG24-5 and SG24-6	VC-E1.46'	2, 3

Safety Related Shock Suppressors (Snubbers)

NOTES:

- (1) Location: AFB - Aux. Boiler Feed Pump Bldg. and Pipe Bridge Area
PAB - Primary Auxiliary Building
VC - Containment Building
SG - Steam Generator

(2) Categories:

1. Snubber in high radiation area during shutdown.*
2. Snubber especially difficult to remove. (Because of size and/or location).
3. Snubber inaccessible during normal operation.* (Because of high radiation and/or temperature environment).
4. Snubber accessible during normal operation.*

* Modifications to this Table due to changes in high radiation areas may be made without prior license amendment provided that a revision to this Table is included with the next license amendment request.

** Deletion of this snubber has been approved in accordance with a redesign of the support system of this line. When conditions permit, this snubber will be removed from its associated piping system and shall then be considered deleted from this Table.

*** Addition of this snubber has been approved in accordance with a redesign of the support system of this line. When conditions permit, this snubber will be added to the appropriate piping system and shall then be considered added to this Table.

4.12 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to the inspection and testing of all hydraulic snubbers listed in Table 3.12-1.

Objective

To verify that snubbers will perform their design functions in the event of a seismic or other transient dynamic event.

Specification

1. All hydraulic snubbers whose seal material has been demonstrated by operating experience, laboratory testing, or analysis to be compatible with the operating environment shall be visually inspected. This inspection shall include, but not necessarily be limited to, inspection of the hydraulic fluid reservoir, fluid connections, and linkage connections to the piping and anchor to verify snubber operability in accordance with the following schedule:

Number of Snubbers Found Inoperable During Inspection or During Inspection Interval	Next Required Inspection Interval
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3,4	124 days \pm 25%
5,6,7	62 days \pm 25%
\geq 8	31 days \pm 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers are categorized in Table 3.12-1 as accessible or inaccessible during reactor operation. These two groups may be inspected independently according to the above schedule.

In addition, if snubber inoperability is identified due to excessive fluid leakage from the external tubing associated with the twenty-four Grinnell snubbers installed at the steam generators, this group of snubbers may be inspected independently according to the above schedule.

2. All hydraulic snubbers whose seal materials have not been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.
3. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule in Specification 4.12.1, it shall be assumed that the facility had been on a 6-month inspection interval.
4. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of proper piston movement, lock-up rate and bleed. For each hydraulic snubber found inoperable, an additional 10% of the remaining hydraulic snubbers or ten hydraulic snubbers, whichever is less, shall be so tested. This procedure shall be repeated until no more failures are found or all hydraulic snubbers subject to the functional testing requirements have been tested. Hydraulic snubbers designated in Table 3.12-1 as Category 1 or Category 2 may be exempted from functional testing provided these snubbers were demonstrated operable during functional testing either at the completion of their fabrication or at a subsequent date. Hydraulic snubbers of rated capacity greater than 50,000 lb shall be exempt from the functional testing requirements.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

By letter dated February 9, 1979, Consolidated Edison Company of New York (the licensee) submitted a proposed amendment to the Technical Specifications for Indian Point Nuclear Generating Plant Unit No. 2. The proposed amendment reduces the number of hydraulic snubbers on two bases: (1) performance of thermal reanalyses, and (2) comparison of similar lines between Indian Point Unit No. 2 and Unit No. 3 and utilizing the results of the dynamic seismic pipe stress analyses for Unit No. 3 lines. By Amendment No. 55, dated June 12, 1979, we approved the proposed reduction of snubbers based on the performance of thermal reanalyses. However, the snubber reduction based on the results of the dynamic seismic pipe stress analyses for Unit No. 3 could not be approved at that time since the Unit No. 3 seismic analyses were being revised. These reanalyses have since been completed and approved by our letter of March 24, 1980 to the Power Authority of the State of New York.

By letter dated December 31, 1979, the licensee modified its February 9, 1979 proposal in four areas:

1. The snubbers approved for deletion by Amendment No. 55 have been removed from the plant and removed from the revised Table 3.12-1. The snubbers that were not removed by Amendment No. 55, but whose removal was originally requested in the February 9, 1979 proposal, are again proposed for deletion.
2. The addition of five snubbers on line No. 361 inside containment to Table 3.12-1.
3. The deletion of one snubber on line No. V-5 in the Auxiliary Feedwater Building outside containment.
4. The revision to paragraph 4.12.1 to permit the inspection of only the Grinnel snubbers if snubber inoperability is caused by leakage from the external tubing.

By letter dated June 23, 1980, the licensee proposed to delete seven snubbers and replace them with passive rigid restraints.

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Evaluation

Item 1 of the presently proposed change was based on system reanalyses by the use of UE&C-ADELPIPE-2 code. The results were accepted by NRC. The deletion of the listed snubbers from Table 3.12-1 is acceptable.

Item 2 proposed the addition of five snubbers to Table 3.12-1. These snubbers were not included in Table 3.12-1 previously. Since their service is required according to the reanalyses, they should be included in Table 3.12-1.

Item 3 proposed the deletion of snubber SR-M38 on line No. V-5 from Table 3.12-1. Since an analysis using the acceptable code UE&C-ADELPIPE-2 proved that this snubber is not required, its deletion from Table 3.12-1 is acceptable.

Item 4 proposed to inspect a group of snubbers independently when the hydraulic fluid of this group of snubbers is supplied by one common reservoir and excessive loss of fluid is identified due to external tubing leakage. Since the external tubing exists only in the unique arrangement whereas a common reservoir supplies hydraulic fluid to a group of snubbers, its leakage will not attribute to a common failure mode for all snubbers in the plant. Therefore, we find this proposal acceptable.

In summary, proposed revisions to Table 3.12-1 in the Technical Specifications for Indian Point Unit No. 2 as described in Items 1, 2 and 3 above are acceptable. Proposed revision to paragraph 4.12.1 in the same Technical Specifications is also acceptable. In addition, the deletion of seven snubbers proposed in the June 23, 1980 letter is based upon the same type of thermal reanalysis that we approved in Amendment No. 55, and is therefore acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date:

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 62 to Facility Operating License No. DPR-26, issued to the Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 2 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective as of the date of issuance.

The amendment reduces the number of hydraulic snubbers required by the Technical Specifications, reflecting redesign of the support systems of certain safety-related pipe lines.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated February 9, 1979 (supplemented by letters dated December 31, 1979 and June 23, 1980), (2) Amendment No. 62 to License No. DPR-26, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of August 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

Joseph D. Neighbors
Joseph D. Neighbors, Acting Chief
Operating Reactors Branch #1
Division of Licensing