

December 7, 1989

Docket No. 50-247

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Mr. Stephen B. Bram
 Vice President, Nuclear Power
 Consolidated Edison Company
 of New York, Inc.
 Broadway and Bleakley Avenue
 Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 73406)

The Commission has issued the enclosed Amendment No. 144 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 12, 1989, as clarified July 11, 1989.

The amendment (1) revises the allowable out-of-service time in Technical Specification (TS) 3.3.B.2.c for inoperable containment spray system valves from 24 hours to 72 hours, (2) adds a new TS 3.3.B.2.d permitting reactor operation to continue for up to 72 hours with the spray additive tank and its associated piping, valves and eductors inoperable provided certain conditions are satisfied, and (3) adds a new TS 3.7.B.5 permitting reactor operation to continue for up to 24 hours with one of the battery chargers associated with station batteries 21, 22, 23 and 24 inoperable provided certain conditions and actions are satisfied.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Donald S. Brinkman, Senior Project Manager
 Project Directorate I-1
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 144 to DPR-26
2. Safety Evaluation

cc: w/enclosures
 See next page

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OFC	:PDI-1	:PDI-1	:OGC	:PDI-1	:SPLB	
NAME	:CVogan	:DBrinkman/bah:	:RCapra	:C.McRAE/KW:		
DATE	: 11/22/89	: 11/22/89	: 11-28/89	: 12/7/89	: 12/5/89	

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 Document Name: AMEND 73406

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Mr. Stephen B. Bram
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 144
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 12, 1989, as clarified July 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

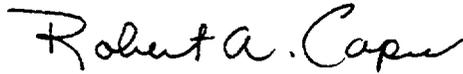
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.144 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and is to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 7, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 144

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

3.3-3
3.7-2

Insert Pages

3.3-3
3.7-2

- e. One channel of heat tracing may be out of service for 48 hours.
 - f. One refueling water storage tank low level alarm may be inoperable for up to 7 days provided the other low level alarm is operable.
3. When RCS temperature is less than or equal to 295°F, the requirements of Table 3.1.A-2 regarding the number of safety injection (SI) pumps allowed to be energized shall be adhered to.

B. Containment Cooling and Iodine Removal Systems

- 1. The reactor shall not be made critical unless the following conditions are met:
 - a. The spray additive tank contains not less than 4000 gallons of solution with a sodium hydroxide concentration of not less than 36% by weight.
 - b. The five fan cooler-charcoal filter units and the two spray pumps, with their associated valves and piping, are operable.
- 2. During power operation, the requirements of 3.3.3.1 may be modified to allow any one of the following components to be inoperable. If the system is not restored to meet the requirements of 3.3.3.1 within the time period specified, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.3.3.1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.
 - a. One fan cooler unit may be inoperable during normal reactor operation for a period not to exceed 7 days provided both containment spray pumps are operable.
 - b. One containment spray pump may be inoperable during normal reactor operation, for a period not to exceed 72 hours, provided the five fan cooler units and the remaining containment spray pump are operable.
 - c. Any valve required for the functioning of the system during and following accident conditions may be inoperable provided it is restored to operable status within 7 days or 72 hours for the fan cooler or containment spray systems respectively, and all valves in the system that provide the duplicate function are operable.
 - d. The spray additive tank and its associated piping, valves and eductors may be inoperable during normal reactor operation for a period not to exceed 72 hours provided both containment spray pumps and the five fan cooler units are operable.

B. During power operation, the following components may be inoperable:

1. Power operation may continue for seven days if one diesel is inoperable provided the 138kv and the 13.8kv sources of off-site power are available and the remaining diesel generators are tested daily to ensure operability and the engineered safety features associated with these diesel generator buses are operable.
2. Power operation may continue for 24 hours, if the 138kv or the 13.8kv source of power is lost, provided the three diesel generators are operable. This operation may be extended beyond 24 hours provided the failure is reported to the NRC within the subsequent 24-hour period with an outline of the plans for restoration of off-site power.
3. If the 138kv power source is lost, in addition to satisfying the requirements of specification 3.7.B.2 above, the -6.9kv bus tie breaker control switches 1-5, 2-5, 3-6, and 4-6 in the CCR shall be placed in the "pull-out" position and tagged to prevent an automatic transfer of the 6.9KV buses 1,2,3 and 4.
4. One battery may be inoperable for 24 hours provided the other batteries and four battery chargers remain operable with one battery charger carrying the dc load of the failed battery's supply system.
5. One battery charger may be inoperable for 24 hours provided the following conditions are satisfied:
 - a. The other three battery chargers and their associated batteries are operable; and
 - b. The affected battery shall have the Specification 4.6.C.1 surveillance initiated within one hour of the time the battery charger is determined to be inoperable and the surveillance shall be repeated every eight hours thereafter to determine battery operability. This surveillance frequency shall be maintained until the battery is declared inoperable or until the battery charger is declared operable.

C. Gas Turbine Generators:

1. At least one gas turbine generator (GT-1, GT-2, or GT-3) and associated switchgear and breakers shall be operable at all times.
2. A minimum of 54,200 gallons of fuel for the operable gas turbine generator shall be available at all times.
3. If the requirements of 3.7.C.1 or 3.7.C.2 cannot be met, then, within the next seven (7) days, either the inoperable condition shall be corrected or an alternate independent power system shall be established.
4. If the requirements of 3.7.C.3 cannot be satisfied, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.7.C.3 cannot be met within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 144 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated June 12, 1989, as clarified July 11, 1989, the Consolidated Edison Company of New York, Inc. (the licensee) requested (1) to increase the allowable out-of-service time in Technical Specification (TS) 3.3.B.2.c for inoperable containment spray system valves from 24 hours to 72 hours, (2) the addition of a new TS 3.3.B.2.d to permit reactor operation to continue for up to 72 hours with the spray additive tank and its associated piping, valves and eductors inoperable provided certain conditions are satisfied, and (3) the addition of a new TS 3.7.B.5 to permit reactor operation to continue for up to 24 hours with one of the battery chargers associated with station batteries 21, 22, 23 and 24 inoperable provided certain conditions and actions are satisfied. The following evaluation addresses the licensee's proposed TS change and additions.

2.0 EVALUATION

Containment Spray System Valves

The proposed change in containment spray system valves allowable out-of-service time would make the valves' allowable out-of-service time consistent with the allowable out-of-service time approved in License Amendment No. 132 (issued on June 29, 1988) for the containment spray system pumps. The proposed change would also make the valves allowable out-of-service time consistent with the Westinghouse Standard Technical Specifications (W-STs). The valves' allowable out-of-service time should have been changed from 24 hours to 72 hours during the processing of License Amendment No. 132; however, due to an administrative error this change was not incorporated in that license amendment. Since the consequences of having the containment spray system inoperable due to an inoperable valve are no different than having the system inoperable due to an inoperable pump and since a 72 hour out-of-service time for the pump was determined acceptable in License Amendment No. 132, the proposed change in allowable out-of-service time for the containment spray system valves is acceptable.

Spray Additive Tank

The proposed addition of TS 3.3.B.2.d would permit reactor operation to continue for up to 72 hours with the spray additive tank and its associated piping, valves and eductors inoperable provided both containment spray pumps

and the five fan cooler units are operable. This proposed additional TS would relieve the licensee from implementing TS 3.0.1 in the event the spray additive tank and its associated piping, valves and eductors are inoperable. TS 3.0.1 requires the licensee to place the reactor in hot shutdown within 7 hours and to be in cold shutdown within the following 30 hours in the event of such inoperability. The requirements of TS 3.0.1 are considered unduly restrictive since a plant-specific PRA evaluation has determined that the proposed 72 hour allowable out-of-service time showed an inconsequential increase in the public health risk. In addition, proposed TS 3.3.B.2.d would be consistent with the W-STs. Therefore, the addition of proposed TS 3.3.B.2.d is acceptable.

Battery Chargers

The proposed addition of TS 3.7.B.5 would permit reactor operation to continue for up to 24 hours with one of the battery chargers associated with station batteries 21, 22, 23 and 24 inoperable provided certain conditions and actions are satisfied. This proposed additional TS would relieve the licensee from implementing TS 3.0.1 in the event a battery charger is inoperable. In addition to providing relief from TS 3.0.1, TS 3.7.B.5 would require verification that the other three battery chargers and their associated batteries are operable as well as prompt verification and frequent reverification that the affected battery remains operable by performing the surveillance requirements of TS 4.6.C.1. Verifying the operability of the other three battery chargers and their associated batteries and performance of the surveillance requirements of TS 4.6.C.1 for the affected battery provides assurance that the DC electrical system remains capable of performing its intended functions. Therefore, the addition of proposed TS 3.7.B.5 is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20. The staff has determined that this amendment involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will

be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 7, 1989

PRINCIPAL CONTRIBUTOR:

D. Brinkman