



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 23, 1994

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Dear Mr. Bram:

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 2 (TAC NO. M89519)

The Commission has issued the enclosed Amendment No. 175 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 29, 1994.

The amendment revises surveillance intervals associated with initiation of Auxiliary Feedwater on steam generator water level (low-low) and on trip of the main feedwater pumps. These revisions are being made in accordance with the guidance provided by Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 175 to DPR-26
2. Safety Evaluation

cc w/encls: See next page

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DFO

Mr. Stephen B. Bram
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station Units 1/2

cc:

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Buchanan, New York 10511

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Senior Resident Inspector
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Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

DATED: September 23, 1994

AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-26-INDIAN POINT UNIT 2

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

C. Miller, 14/A/4

M. J. Case

C. Vogan

F. Williams

OGC

D. Hagan, 3302 MNBB

G. Hill (2), P1-22

C. Grimes, 11/F/23

ACRS (10)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated April 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 175, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael J. Case, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 23, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

Table 4.1-1 (page 4 of 7)

Table 4.1-1 (page 5 of 7)

Insert Pages

Table 4.1-1 (page 4 of 7)

Table 4.1-1 (page 5 of 7)

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and
Tests of Instrument Channels

Channel Description	Check	Calibrate	Test	Remarks
27. Turbine Trip				
a. Low Auto Stop Oil Pressure	N.A.	R	N.A.	
28. Control Rod Protection (for use with LOPAR fuel)	N.A.	R	*	
29. Loss of Power				
a. 480v Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	R	
b. 480v Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	R	
c. 480v Emergency Bus Undervoltage (Alarm)	N.A.	R	M	
30. Auxiliary Feedwater				
a. Steam Generator Water Level (Low-Low)	S	R#	R#	
b. Low-Low Level AFWS Automatic Actuation Logic	N.A.	N.A.	M	Test one logic channel per month on an alternating basis.
c. Station Blackout (Undervoltage)	N.A.	R	R	

* Within 31 days prior to entering a condition in which the Control Rod Protection System is required to be operable unless the reactor trip breakers are manually opened during RCS cooldown prior to T_{cold} decreasing below 350°F and the breakers are maintained opened during RCS cooldown when T_{cold} is less than 350°F.

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and
Tests of Instrument Channels

Channel Description	Check	Calibrate	Test	Remarks
d. Trip of Main Feedwater Pumps	N.A.	N.A.	R#	
31. Reactor Coolant System Subcooling Margin Monitor	M	R	N.A.	
32. PORV Position Indicator (Limit Switch)	M	R#	R#	
33. PORV Block Valve Position Indicator (Limit Switch)	M*	R#	R#	
34. Safety Valve Position Indicator (Acoustic Monitor)	M	R#	R#	
35. Auxiliary Feedwater Flow Rate	M	R	R	
36. PORV Actuation/ Reclosure Setpoints	N.A.	R#	N.A.	
37. Overpressure Protection System (OPS)	N.A.	R#	**	

* Except when block valve operator is deenergized.

** Within 31 days prior to entering a condition in which OPS is required to be operable and at monthly intervals thereafter when OPS is required to be operable.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated April 29, 1994, Consolidated Edison Company of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications (TSs). The requested changes are a follow-up to License Amendment No. 159, issued on December 10, 1992, which changed the TS Section 1.0, Definitions, to accommodate a 24-month fuel cycle and which extended test intervals for specific surveillance tests. The requested changes in this proposal would extend the surveillance intervals associated with initiation of auxiliary feedwater on steam generator water level (low-low) and on trip of the main feedwater pumps. These revisions are being made in accordance with the guidance provided by Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle."

It is noted that the licensee submitted an amendment request on November 25, 1992, for extending the test interval for the Auxiliary Feedwater System, TS Section 4.8, which was granted by Amendment No. 166 issued on December 16, 1993. The licensee determined, however, during subsequent audits that two other TS locations associated with the Auxiliary Feedwater System were not identified as requiring revision. This amendment request is for the extension of those surveillances. The evaluation provided for this request is consistent with that issued with Amendment No. 166.

2.0 EVALUATION

The Auxiliary Feedwater System supplies high-pressure feedwater to the steam generators to maintain a water inventory for decay heat removal. The system is used for normal startup and shutdown as well as for events leading to a loss of main feedwater. The refueling interval surveillance requires verification of proper operation of the initiating logic for the auxiliary feedwater system components upon receipt of test signals for each mode of operation. The licensee reviewed completed test procedures for the last four refuelings which covered a period in excess of 5 years. In one instance a test result did not meet the acceptance criteria. In 1986, a relay failed to pickup due to dirty contacts. This failure only affected the steam driven

auxiliary feedwater pump and would not have prevented the auxiliary feedwater system from supplying more than the required amount of feedwater. The licensee concluded that no significant increase in probability or consequence of an accident would be incurred by extending the surveillance interval.

The licensee has evaluated the effect of the increase in the above surveillance intervals on safety and has concluded that the effect is small. The licensee has confirmed that historical plant maintenance and surveillance data do not invalidate this conclusion. The licensee also confirmed that the increase in surveillance intervals to accommodate a 24-month fuel cycle does not invalidate any assumption in the plant licensing basis. The staff reviewed this information and agrees with the conclusions contained therein. The staff finds that the proposed TS changes do not have a significant effect on safety and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 42335). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Francis J. Williams

Date: September 23, 1994

September 23, 1994

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
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Broadway and Bleakley Avenue
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Sincerely,
Original signed by:
Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 175 to DPR-26
2. Safety Evaluation

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