

December 8, 1994

Mr. Stephen E. Quinn
Vice President, Nuclear Power
Consolidated Edison Company of
New York
Broadway and Bleakley Avenue
Buchanan, NY 10511

SUBJECT: ADMINISTRATIVE ERROR IN ISSUANCE OF AMENDMENT NO. 173, INDIAN
POINT NUCLEAR GENERATING UNIT NO. 2 (TAC NO. M88038)

Dear Mr. Quinn:

By letter dated November 8, 1994, you advised us that Amendment No. 173, which was issued on July 26, 1994, and which addressed Technical Specification (TS) changes to remove cycle-specific parameter limits and to reference a Core Operating Limits Report containing these limits, included page 6-27, which should have been resubmitted when it was revised by Amendment No. 171 which was issued on July 7, 1994. You submitted a corrected page 6-27. This change is administrative and does not include any changes not previously evaluated. In addition, you included a revised Table of Contents for the TS. We are enclosing the corrected page 6-27, and the revised Table of Contents for substitution and distribution.

Sincerely,
Original signed by

Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9412150058 941208
FDR ADOCK 05000247
P FDR

Enclosures: As stated

cc w/encls: See next page

Distribution:

Docket File CVogan ACRS (4)
PUBLIC FWilliams OPA
PDI-1 Reading OGC OC/LFDCB
SVarga G. Hill, IRM (2) CCowgill, RGN-I
JZwolinski 120026 C. Grimes, DOPS/OTSB
MCase

DOCUMENT NAME: G:\IP2\IP288038.LTR

To receive a copy of this document, indicate in the box: "C" = Copy without enclosures "E" = Copy with enclosures "N" = No copy

OFFICE	LA:PDI-1	C	PM:PDI-1	E	D:PDI-1				
NAME	CVogan	CV	FWilliams	Lans	smm	MCase			
DATE	12/5/94		12/6/94		12/6/94		12/ /94		12/ /94

OFFICIAL RECORD COPY

NRC FILE CENTER COPY

CP1

DFO1



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 8, 1994

Mr. Stephen E. Quinn
Vice President, Nuclear Power
Consolidated Edison Company of
New York
Broadway and Bleakley Avenue
Buchanan, NY 10511

SUBJECT: ADMINISTRATIVE ERROR IN ISSUANCE OF AMENDMENT NO. 173, INDIAN
POINT NUCLEAR GENERATING UNIT NO. 2 (TAC NO. M88038)

Dear Mr. Quinn:

By letter dated November 8, 1994, you advised us that Amendment No. 173, which was issued on July 26, 1994, and which addressed Technical Specification (TS) changes to remove cycle-specific parameter limits and to reference a Core Operating Limits Report containing these limits, included page 6-27, which should have been resubmitted when it was revised by Amendment No. 171 which was issued on July 7, 1994. You submitted a corrected page 6-27. This change is administrative and does not include any changes not previously evaluated. In addition, you included a revised Table of Contents for the TS. We are enclosing the corrected page 6-27, and the revised Table of Contents for substitution and distribution.

Sincerely,

A handwritten signature in cursive script, appearing to read "Francis J. Williams, Jr.", written in dark ink.

Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/encls: See next page

Stephen E. Quinn
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station Units 1/2

cc:

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, NY 12223

Mr. Charles W. Jackson
Manager of Nuclear Safety and
Licensing
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P.O. Box 38
Buchanan, NY 10511

Mr. Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, NY 10003

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Mr. Peter Kokolakis, Director
Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Mr. Walter Stein
Secretary - NFSC
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, NY 10003

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

TECHNICAL SPECIFICATIONS

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
1.0	Definitions	
1.1	a. Rated Power	1-1
	b. Thermal Power	1-1
1.2	Reactor Operating Conditions	1-1
1.3	Operable-Operability	1-1
1.4	Protective Instrumentation Logic	1-2
1.5	Degree of Redundancy	1-2
1.6	Instrumentation Surveillance	1-2
1.7	Containment Integrity	1-3
1.8	Quadrant Power Tilt Ratio	1-3
1.9	Surveillance Intervals	1-4
1.10	Deleted	1-4
1.11	Pressure Boundary Leakage	1-4
1.12	Identified Leakage	1-5
1.13	Unidentified Leakage	1-5
1.14	Dose Equivalent I-131	1-5
1.15	Gaseous Radwaste Treatment System	1-6
1.16	Member(s) of the Public	1-6
1.17	Offsite Dose Calculation Manual (ODCM)	1-6
1.18	Process Control Program (PCP)	1-6
1.19	Purge-Purging	1-6
1.20	Site Boundary	1-6
1.21	Solidification	1-7
1.22	Unrestricted Area	1-7
1.23	Ventilation Exhaust Treatment System	1-7
1.24	Venting	1-7
1.25	Core Operating Limits Report (COLR)	1-8
2.0	Safety Limits and Limiting Safety System Settings	2.1-1
2.1	Safety Limit: Reactor Core	2.1-1
2.2	Safety Limit: Reactor Coolant System Pressure	2.2-1
2.3	Limiting Safety System Settings, Protective Instrumentation	2.3-1
3.0	Limiting Conditions for Operation	3.1.A-1
3.1	Reactor Coolant System	3.1.A-1
	A. Operational Components	3.1.A-2
	B. Heatup and Cooldown	3.1.B-1
	C. Minimum Conditions for Criticality	3.1.C-1
	D. Maximum Reactor Coolant Activity	3.1.D-1
	E. Maximum Reactor Coolant Oxygen, Chloride and Fluoride Concentration	3.1.E-1
	F. Reactor Coolant System Leakage and Leakage into the Containment Free Volume	3.1.F-1
	G. Reactor Coolant System Pressure, Temperature and Flow Rate	3.1.G-1
3.2	Chemical and Volume Control System	3.2-1

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
3.3	Engineered Safety Features	3.3-1
	A. Safety Injection and Residual Heat Removal Systems	3.3-1
	B. Containment Cooling and Iodine Removal Systems	3.3-3
	C. Isolation Valve Seal Water System (IVSWS)	3.3-5
	D. Weld Channel and Penetration Pressurization System (WC&PPS)	3.3-6
	E. Component Cooling System	3.3-7
	F. Service Water System	3.3-8
	G. Hydrogen Recombiner System and Post-Accident Containment Venting System	3.3-10
	H. Control Room Air Filtration System	3.3-11
	I. Cable Tunnel Ventilation Fans	3.3-12
3.4	Steam and Power Conversion System	3.4-1
3.5	Instrumentation Systems	3.5-1
3.6	Containment System	3.6-1
	A. Containment Integrity	3.6-1
	B. Internal Pressure	3.6-3
	C. Containment Temperature	3.6-3
3.7	Auxiliary Electrical Systems	3.7-1
3.8	Refueling, Fuel Storage and Operations with the Reactor Vessel Head Bolts Less Than Fully Tensioned	3.8-1
3.9	Radioactive Effluents	3.9-1
	A. Radioactive Liquid Effluents	3.9-1
	B. Radioactive Gaseous Effluents	3.9-4
	C. Uranium Fuel Cycle Dose Commitment	3.9-9
	D. Solid Radioactive Waste	3.9-10
3.10	Control Rod and Power Distribution Limits	3.10-1
	3.10.1 Shutdown Reactivity	3.10-1
	3.10.2 Power Distribution Limits	3.10-1
	3.10.3 Quadrant Power Tilt Limits	3.10-4
	3.10.4 Rod Insertion Limits	3.10-5
	3.10.5 Rod Misalignment Limitations	3.10-6
	3.10.6 Inoperable Rod Position Indicator Channels	3.10-7
	3.10.7 Inoperable Rod Limitations	3.10-7
	3.10.8 Rod Drop Time	3.10-8
	3.10.9 Rod Position Monitor	3.10-8
	3.10.10 Quadrant Power Tilt Monitor	3.10-8
3.11	Movable Incore Instrumentation	3.11-1
3.12	Shock Suppressors (Snubbers)	3.12-1
3.13	Fire Protection and Detection Systems	3.13-1
	A. High-Pressure Water Fire Protection System	3.13-1
	B. Fire Protection Spray Systems	3.13-3
	C. Penetration Fire Barriers	3.13-3
	D. Fire Detection Systems	3.13-4
	E. Fire Hose Stations and Hydrants	3.13-5
	F. Cable Spreading Room Halon System	3.13-5
3.14	Hurricane Alert	3.14-1
3.15	Meteorological Monitoring System	3.15-1
3.16	Reactor Coolant System Vents	3.16-1

TABLE OF CONTENTS (Cont'd)

<u>Section</u>	<u>Title</u>	<u>Page</u>
6.5.2	Nuclear Facilities Safety Committee (NFSC)	
6.6	Reportable Event Action	6-8
6.7	Safety Limit Violation	6-13
6.8	Procedures and Programs	6-14
6.9	Reporting Requirements	6-14
	Startup Report	6-16
	Annual Radiation Exposure Report	6-16
	Annual Radiological Environmental Operating Report	6-17
	Annual Radioactive Effluent Report	6-18
	Monthly Operating Report	6-19
	Core Operating Limits Report	6-21
	Special Reports	6-22
6.10	Record Retention	6-23
6.11	Radiation Protection Program	6-24
6.12	High Radiation Area	6-26
6.13	Environmental Qualification	6-27
6.14	Process Control Program (PCP)	6-27
6.15	Offsite Dose Calculation Manual (ODCM)	6-28
6.16	Major Changes to Radioactive Liquid, Gaseous and Solid Waste Systems	6-29

6.12 HIGH RADIATION AREA

6.12.1 As an acceptable alternative to the "control device" or "alarm signal" required by 10 CFR 20.203(c)(2):

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of Specification 6.12.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry to such areas and the keys shall be maintained under the administrative control of the Radiation Protection Manager and/or the Senior Watch Supervisor on duty.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines), or NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of License No. DPR-26 dated October 24, 1980.