

August 8, 1985

Docket No. 50-286

<u>Distribution</u>	<u>Docket file</u>
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Mr. John C. Brons  
 Senior Vice President - Nuclear Generation  
 Power Authority of the State of New York  
 123 Main Street  
 White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No.60 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated October 31, 1984.

The amendment modifies the function and membership of the Safety Review Committee (SRC) of the Power Authority of the State of New York for Indian Point 3.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,  
 /s/JDNeighbors

Joseph D. Neighbors, Project Manager  
 Operating Reactors Branch #1  
 Division of Licensing

Enclosures:

1. Amendment No. 60 to DPR-64
2. Safety Evaluation

cc: w/enclosures  
 See next page

ORB#1:DL  
 CParrish *cp*  
 06/14/85

ORB#1:DL *del*  
 KJohnston/ts  
 06/17/85

ORB#1:DL  
 DNeighbors  
 06/17/85

BC-ORB#1:DL  
 SVarga  
 06/17/85

OELD  
 Jim  
 06/20/85

AD-OR:DL  
 GLainas  
 06/28/85

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Mr. John C. Brons  
Power Authority of the State  
of New York

Indian Point 3 Nuclear  
Generating

cc:  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Resident Inspector  
Indian Point Nuclear Generating  
U.S. Nuclear Regulatory Commission  
Post Office Box 66  
Buchanan, New York 10511

Mr. Gerald C. Goldstein  
Assistant General Counsel  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019

Mr. Robert L. Spring  
Nuclear Licensing Engineer  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place  
New York, New York 10003

Ms. Ellyn Weiss  
Harmon, Weiss and Jordan  
2001 S Street, N.W., Suite 430  
Washington, DC 20009

Mr. A. Klausmann, Vice President  
Quality Assurance  
Power Authority of the State  
of New York  
10 Columbus Circle  
New York, New York 10019

Dr. Lawrence R. Quarles  
Apartment 51  
Kendal at Longwood  
Kennett Square, Pennsylvania 19348

Mayor, Village of Buchanan  
236 Tate Avenue  
Buchanan, New York 10511

Mr. George M. Wilverding, Manager  
Nuclear Safety Evaluation  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, New York 10601

Mr. F. X. Pindar  
Quality Assurance Superintendent  
Indian Point 3 Nuclear Power Plant  
Post Office Box 215  
Buchanan, New York 10511

Director, Technical Development  
Programs  
State of New York Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

S. S. Zulla, Vice President  
Nuclear Support  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, New York 10601

Mr. Leroy W. Sinclair  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, New York 10601

Mr. William Josiger  
Resident Manager  
Indian Point 3 Nuclear Power Plant  
Post Office Box 215  
Buchanan, New York 10511

Power Authority of the State  
of New York

- 2 -

Indian Point 3

cc

Ezra I. Bialik  
Assistant Attorney General  
Environmental Protection Bureau  
New York State Department of Law  
2 World Trade Center  
New York, New York 10047

P. Kokolakis, Director  
Nuclear Licensing  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, New York 10601

Mr. Jay Dunkleberger  
Division of Policy Analysis  
and Planning  
New York State Energy Office  
Agency Building 2, Empire  
State Plaza  
Albany, New York 12223



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated October 31, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

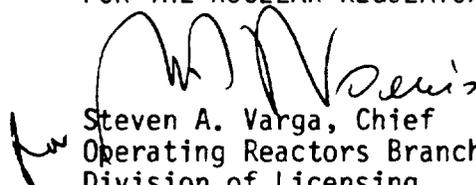
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 8, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

6-8  
6-9

Insert Pages

6-8  
6-9

RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and Executive Vice President-Nuclear Generation.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Emergency Planning
- m. Nuclear Licensing
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant.

## MEMBERSHIP

6.5.2.2. The SRC shall be composed of the following voting members:

Chairman:	Manager-Nuclear Safety Evaluation
Vice-Chairman:	Director-Quality Assurance
Member:	Vice President Nuclear Support - BWR
Member:	Vice President Nuclear Support - PWR
Member:	Vice President - Generic Nuclear Support
Member:	Vice President - Design and Analysis
Member:	Resident Manager - IP3
Member:	Resident Manager - JAF
Member:	Consultant

## ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman. An Alternate Vice-Chairman shall be appointed in writing by the Executive Vice President-Nuclear Generation if necessary. However, no more than two alternates shall participate as voting members in SRC activities at any one time.

## CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

## MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

## QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman or Alternate Vice-Chairman and four members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

Introduction

The Power Authority of the State of New York (PASNY) has requested, by letter dated October 31, 1984, changes to the Administrative Controls Section of the Technical Specifications for the Indian Point 3 Nuclear Power Plant. Below is a brief description and our evaluation of the changes.

Discussion

1. The Safety Review Committee (SRC) membership is to be revised to include the next-to-highest level management in the Nuclear Generation, Engineering and Quality Assurance, and Reliability Departments. The existing and the proposed membership of the SRC are shown below.

Present:

Chairman:	Manager-Nuclear Safety Evaluation
Vice-Chairman:	Vice President-Quality Assurance
Member:	Vice President Nuclear Support-BWR
Member:	Vice President Nuclear Support-PWR
Member:	Manager-Radiological Health and Chemistry
Member:	Director-Nuclear Design and Analysis
Member:	Director-Electrical Design and Analysis
Member:	Director of Environmental Programs
Member:	Director-Civil/Structural Design and Analysis
Member:	Director-Mechanical Design and Analysis
Member:	Director-Piping and Process-Design and Analysis
Member:	Manager Operational Analysis and Training

Proposed:

Chairman:	Manager-Nuclear Safety Evaluation
Vice-Chairman:	Director-Quality Assurance
Member:	Vice President Nuclear Support - BWR
Member:	Vice President Nuclear Support - PWR
Member:	Vice President - Generic Nuclear Support
Member:	Vice President - Design and Analysis
Member:	Resident Manager - IP3
Member:	Resident Manager - JAF
Member:	Consultant

This change in the committee membership reduces the committee to nine (9) voting members. Title changes have been made throughout Section 6 to reflect the new membership.

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2. The SRC's function described in Section 6.5.2.1 has been modified to be consistent with NUREG-0452, Revision 3, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors." The specific change is shown below:

Present:

6.5.2.1 The SRC shall collectively have the competence required to review problems in the following areas:

Proposed:

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

Evaluation

During its review of the changes in Item 1 of the Discussion, the staff considered the safety implications of the removal of the Design and Analysis Directors from the membership of the Safety Review Committee (SRC). For the following reasons, it was determined that no measurable decrease in the effectiveness of the SRC would occur.

1. The qualifications of the Vice President level managers to be included on the SRC are: a BS in Engineering of Science and six years of professional level experience in nuclear services, nuclear plant operations, or nuclear engineering, and the necessary overall nuclear background to determine when to call consultants and contractors for dealing with complex problems beyond the scope of the owner-organization expertise. The qualifications of the Directors removed from the SRC are: a BS in Engineering or Science and three years in design, operation or other related activities of nuclear power plants.

The experience level of the VPs in actual operations service or engineering of nuclear power plants is therefore acceptable.

2. The reporting responsibilities of the SRC Chairman are not affected by the SRC membership changes. The Chairman shall continue to report directly to the Executive Vice President-Nuclear Generation.
3. The implementation of the SRC charter usually relies on outside organizations or the Technical Divisions to perform the actual review and/or audit for which the SRC is responsible. Therefore, although the Technical Division Directors were removed from the SRC membership, they will, in essence, continue to remain involved with the SRC review process. Moreover, the Directors replaced from the Engineering and Generation Departments will be the alternates to the new VP level voting members from their same departments.
4. The SRC is an oversight body which ordinarily meets every six months to review actions and reports that have already been implemented or accomplished. In most cases, the Plant Operations Review Committee (PORC) and/or the Independent Safety Engineering Group (ISEG) have previously performed reviews of the same actions being reviewed by the SRC.

5. The addition of the consultant as a voting member was considered a strength and could bring specific expertise to be used in the resolution of problems, if required.
6. The reduction in the number of SRC membership from 12 to 9 was considered a strength. The SRC should be more effective in bringing about closure of any identified safety issues with the smaller number of members and the increased level of decision-making capability.
7. The increased participation of upper management attention and involvement in the safety review process should aid in achieving safe plant operation.

The changes described in Items 1 and 2 of the Discussion above are in accordance with the guidance provided in Section 13.4 of the Standard Review Plan, NUREG-0800 and the Standard Technical Specifications for Westinghouse Pressurized Water Reactors, NUREG-0452. Accordingly, the proposed changes are acceptable.

#### Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 8, 1985

Principal Contributor:

P. W. McLaughlin