

June 5, 1989

Docket No. 50-247

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Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 69540)

The Commission has issued the enclosed Amendment No. 141 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 23, 1988.

The amendment adds a surveillance requirement to the Technical Specifications requiring periodic testing of the backup nitrogen supply system for operating the pressurizer power operated relief valves.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 141 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

[AMEND 247 TAC NO. 69540]

OFC : PDI-I	: PDI-I	: OGC	: PDI-I	:	:	:
NAME : CVogan	: DBrinkman	: vr	: RCapra	:	:	:
DATE : 5/11/89	: 5/15/89	: 5/30/89	: 6/5/89	:	:	:

Handwritten notes: "OK wanted revision" above OGC; "OK" above DBrinkman; "RC" above RCapra.

OFFICIAL RECORD COPY

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PDR ADOCK 05000247
P FDC

Handwritten notes: "DFD 11" and "c/p-1" with initials.

Mr. Stephen B. Bram
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.141
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated September 23, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 141, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective no later than 30 days after its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 5, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 141

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

4.18-1

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Insert Pages

4.18-1

4.18-2 (new page)

4.18 OVERPRESSURE PROTECTION SYSTEM

Applicability

This specification applies to the surveillance requirements for the OPS provided for prevention of RCS overpressurization.

Objective

To verify the operability of OPS.

Specifications

- A. When the OPS PORVs are being used for overpressure protection as required by Specification 3.1.A.4, their associated series MOVs shall be verified to be open at least twice weekly with a maximum time between checks of 5 days.
- B. When RCS venting is being used for overpressure protection as permitted by Specification 3.1.A.4, the vent(s) shall be verified to be open at least daily. When the venting pathway is provided with a valve which is locked, sealed, or otherwise secured in the open position, then only these valves need be verified to be open at monthly intervals.
- C. When pressurizer pressure and level control is being used for overpressure protection, as permitted by Specification 3.1.A.4, then these parameters shall be verified to be within their limits at least once per shift.
- D. When safety injection pumps and/or charging pumps are required to be de-energized per Specification 3.1.A.4, the pumps shall be demonstrated to be inoperable at monthly intervals by verifying lockout of the pump circuit breakers at the 480 volt switchgear, or once per shift if other means of de-energizing the pumps are used.
- E. The PORV backup nitrogen system shall be demonstrated to be operable at refueling surveillance intervals.

Basis

These specifications establish the surveillance program for the Overpressure Protection System (OPS). This surveillance program is intended to verify

the operability of the system and will identify for corrective action any conditions which could prevent any portion of the system from performing its intended function.

The PORVs and MOVs associated with the OPS are not included in this specification since the valve cycling and operability tests for these valves are performed in accordance with applicable testing requirements of the ASME Code Section XI and 10 CFR 50.55a.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 141 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

INTRODUCTION

By letter dated September 23, 1988, the Consolidated Edison Company of New York, Inc. (the licensee) requested that a new Technical Specification (TS) 4.18.E be added to require a surveillance test of the backup nitrogen supply system for operating the pressurizer power operated relief valves. The following evaluation addresses the licensee's proposed TS change.

EVALUATION

The licensee proposed adding a new TS which would require a periodic demonstration of the operability of PORV backup nitrogen system at refueling surveillance intervals. The proposed surveillance test would require a demonstration of the capability of the PORV backup nitrogen system to operate the PORVs if the normal nitrogen supply system is not available.

The normal nitrogen system is continuously pressurized during normal operation. The makeup capacity of the normal nitrogen system would mask potential leakage from the backup nitrogen system and therefore leakage from the backup system may not be readily apparent during normal operation. Also, the capability of the backup nitrogen system to isolate itself (via check valves) from the normal nitrogen system in the event the normal nitrogen system loses pressure cannot be readily demonstrated during normal operation. Therefore, the staff concludes that it is appropriate to perform a periodic surveillance test of the backup nitrogen system to demonstrate its integrity. The staff also concludes that this demonstration of the operability at refueling intervals is the appropriate test frequency since the backup nitrogen system cannot be readily tested during normal operation.

The proposed surveillance test would not require cycling of the PORV as part of this surveillance test since the PORV is routinely tested (including cycling) during tests performed in accordance with the inservice test requirements of Section XI of the ASME Boiler and Pressure Vessel Code. Therefore, the staff finds the proposed Technical Specification acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20 and changes a surveillance requirement. The staff has determined that this amendment involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure.

The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 5, 1989

PRINCIPAL CONTRIBUTOR:

D. S. Brinkman